



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 13, 1995

Mr. William J. Cahill, Jr.
Chief Nuclear Officer
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER
PLANT (TAC NO. M92039)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 229 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated April 12, 1995.

The amendment extends the surveillance test intervals for the nuclear steam supply system to support 24-month operating cycles. Surveillance test interval extensions that are justified will be denoted as being performed "every 24 months" or "at least once per 24 months" consistent with the guidance provided in Reference 1. Other surveillances currently performed "once each operating cycle," "at least once during each operating cycle," "each refueling," or similar notation, that are not being extended at this time will be denoted as being performed "at least once per 18 months." The NRC staff has determined that the proposed TS changes follow the guidance of Generic Letter 91-04, and are therefore acceptable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

C. E. Carpenter, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket No. 50-333

Enclosures: 1. Amendment No. 229 to DPR-59
2. Safety Evaluation

cc w/encls: See next page

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James A. FitzPatrick Nuclear
Power Plant

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DATED: October 13, 1995

AMENDMENT NO. 229 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 229
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 12, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 229, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 13, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 229

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

86a
119
142a
143
152

Insert Pages

86a
119
142a
143
152

JAFNPP

TABLE 4.2-8 (cont'd)

**MINIMUM TEST AND CALIBRATION FREQUENCY FOR
ACCIDENT MONITORING INSTRUMENTATION**

Instrument	Instrument Functional Test	Calibration Frequency	Instrument Check
15. Core Spray Flow	N/A	Once/Operating Cycle	Once/day
16. Core Spray Discharge Pressure	N/A	Once/Operating Cycle	Once/day
17. LPCI (RHR) Flow	N/A	Once/Operating Cycle	Once/day
18. RHR Service Water Flow	N/A	Once/Operating Cycle	Once/day
19. Safety/Relief Valve Position Indicator (Primary and Secondary)	Once/24 months	N/A	Once/month
20. Torus Water Level (narrow range)	N/A	Once/Operating Cycle	Once/day
21. Drywell-Torus Differential Pressure	N/A	Once/Operating Cycle	Once/day

JAFNPP

3.5 (cont'd)

D. Automatic Depressurization System (ADS)

1. The ADS shall be operable with at least 5 of the 7 ADS valves operable:
 - a. whenever the reactor pressure is greater than 100 psig and irradiated fuel is in the reactor vessel, and
 - b. prior to reactor startup from a cold condition.

4.5 (cont'd)

D. Automatic Depressurization System (ADS)

1. Surveillance of the Automatic Depressurization System shall be performed at least once every 24 months as follows:
 - a. A simulated automatic actuation which opens all pilot valves.
 - b. A simulated automatic actuation which is inhibited by the override switches.

JAFNPP

3.6 (cont'd)

E. Safety/Relief Valves

1. During reactor power operating conditions and prior to startup from a cold condition, or whenever reactor coolant pressure is greater than atmosphere and temperature greater than 212 F, the safety mode of at least 9 of 11 safety/relief valves shall be operable. The Automatic Depressurization System valves shall be operable as required by specification 3.5.D.

4.6 (cont'd)

E. Safety/Relief Valves

1. At least 5 of the 11 safety/relief valves shall be bench checked or replaced with bench checked valves every 24 months. All valves shall be tested every 48 months. The testing shall demonstrate that each valve tested actuates at 1110 psig $\pm 3\%$. Following testing, lift settings shall be 1110 psig $\pm 1\%$.

JAFNPP

3.6 (cont'd)

2. If Specification 3.6.E.1 is not met, the reactor shall be placed in a cold condition within 24 hours.
3. Low power physics testing and reactor operator training shall be permitted with inoperable components as specified in Specification 3.6.E.1 above, provided that reactor coolant temperature is ≤ 212 F and the reactor vessel is vented or the reactor vessel head is removed.
4. The provisions of Specification 3.0.D are not applicable.
5. The safety and safety/relief valves are not required to be operable during hydrostatic pressure and leakage testing with reactor coolant temperatures between 212 F and 300 F and irradiated fuel in the reactor vessel provided all control rods are inserted.

4.6 (cont'd)

2. At least one safety/relief valve shall be disassembled and inspected every 24 months.
3. The integrity of the nitrogen system and components which provide manual and ADS actuation of the safety/relief valves shall be demonstrated at least once every 3 months.
4. Manually open each safety/relief valve while bypassing steam to the condenser and observe a $\geq 10\%$ closure of the turbine bypass valves, to verify that the safety/relief valve has opened. This test shall be performed at least every 24 months while in the RUN mode and within the first 12 hours after steam pressure and flow are adequate to perform the test.

3.6 and 4.6 BASES (cont'd)

E. Safety/Relief Valves

The safety/relief valves (SRVs) have two modes of operation; the safety mode or the relief mode. In the safety mode (or spring mode of operation) the spring loaded pilot valve opens when the steam pressure at the valve inlet overcomes the spring force holding the pilot valve closed. The safety mode of operation is required during pressurization transients to ensure vessel pressures do not exceed the reactor coolant pressure safety limit of 1,375 psig.

In the relief mode the spring loaded pilot valve opens when the spring force is overcome by nitrogen pressure which is provided to the valve through a solenoid operated valve. The solenoid operated valve is actuated by the ADS logic system (for those SRVs which are included in the ADS) or manually by the operator from a control switch in the main control room or at the remote ADS panel. Operation of the SRVs in the relief mode for the ADS is discussed in the Bases for Specification 3.5.D.

Experiences in safety/relief valve testing have shown that failure or deterioration of safety/relief valves can be adequately detected if at least 5 of the 11 valves are bench tested once every 24 months so that all valves are tested every 48 months. Furthermore, safety/relief valve testing experience has demonstrated that safety/relief valves which actuate within $\pm 3\%$ of the design pressure setpoint are considered operable (see ANSI/ASME OM-1-1981). The safety bases for a single nominal valve opening pressure of 1110 psig are described in NEDC-31697P, "Updated SRV Performance Requirements for the JAFNPP." The single nominal setpoint is set below the reactor vessel design pressure (1250 psig) per the requirements of Article 9 of the ASME Code - Section III, Nuclear Vessels. The setting of 1110 psig preserves the safety margins associated

with the HPCI and RCIC turbine overspeed systems and the Mark I torus loading analyses. Based on safety/relief valve testing experience and the analysis referenced above, the safety/relief valves are bench tested to demonstrate that in-service opening pressures are within the nominal pressure setpoints $\pm 3\%$ and then the valves are returned to service with opening pressures at the nominal setpoints $\pm 1\%$. In this manner, valve integrity is maintained from cycle to cycle.

The analyses with NEDC-31697P also provide the safety basis for which 2 SRVs are permitted inoperable during continuous power operation. With more than 2 SRVs inoperable, the margin to the reactor vessel pressure safety limit is significantly reduced, therefore, the plant must enter a cold condition within 24 hours once more than 2 SRVs are determined to be inoperable. (See reload evaluation for the current cycle).

A manual actuation of each SRV is performed to demonstrate that the valves are mechanically functional and that no blockage exists in the valve discharge line. Valve opening is confirmed by monitoring the response of the turbine bypass valves and the SRV acoustic monitors. Adequate reactor steam dome pressure must be available to avoid damaging the valve. Adequate steam flow is required to ensure that reactor pressure can be maintained during the test. Testing is performed in the RUN mode to reduce the risk of a reactor scram in response to small pressure fluctuations which may occur while opening and reclosing the valves.

Low power physics testing and reactor operator training with inoperable components will be conducted only when the safety/relief valves are



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 229 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated April 12, 1995, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant (FitzPatrick) Technical Specifications (TSs). The requested changes would extend the surveillance test intervals for the nuclear steam supply system (NSSS) to support 24-month operating cycles.

Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate 24-Month Fuel Cycle," dated April 2, 1991, provides generic guidance to support the development of TS revisions to allow a 24-month fuel cycle and includes requirements to evaluate the effect on safety for an increase in surveillance intervals to accommodate a 24-month cycle.

The specific surveillances that were requested to be revised by this application are:

1. Page 86a, Item 19, Table 4.2-8, "Minimum Test and Calibration Frequency for Accident Monitoring Instrumentation," change the instrument functional test surveillance frequency of the safety/relief valve position indicator from "once/operating cycle" to "once/24 months."
2. Page 119, Specification 4.5.D.1, change the Automatic Depressurization System (ADS) surveillances that are performed "during each operating cycle" to "at least once every 24 months." In addition, parts a and b of this specification are further clarified by changing "initiation" to "actuation" to better conform to the definition of "Simulated Automatic Actuation" in Section 1.0. The revised specification reads:

"Surveillance of the Automatic Depressurization System shall be performed every 24 months as follows:

- a. A simulated automatic actuation which opens all pilot valves.
- b. A simulated automatic actuation which is inhibited by the override switches."

3. Page 142a, Specification 4.6.E.1, change the safety/relief valve bench check from "once per operating cycle" to "every 24 months." In the second sentence, change test frequency for all valves from "every two operating cycles.*" to "every 48 months." The revised specification reads:

"At least 5 of the 11 safety/relief valves shall be bench checked or replaced with bench checked valves every 24 months. All valves shall be tested every 48 months."

4. Page 143, Specification 4.6.E.2, change the safety/relief valve disassembly and inspection from "once/operating cycle.*" to "every 24 months." The revised specification reads:

"At least one safety/relief valve shall be disassembled and inspected every 24 months."

5. Page 143, Specification 4.6.E.4, change test frequency to manually open each safety/relief valve from "each operating cycle" to "every 24 months." The revised specification reads:

"This test shall be performed at least once every 24 months while in the RUN mode and within the first 12 hours after steam pressure and flow are adequate to perform this test."

In addition, the asterisk and footnote from pages 142a and 143 are deleted since the conditions for surveillance test extension have expired.

6. Bases Page 152, Section 4.6, change bench testing requirements (third paragraph) from "once per operating cycle so that all valves are tested every two operating cycles" to "every 24 months so that all valves are tested every 48 months." The revised bases reads:

"Experiences in safety/relief valve testing have shown that failure or deterioration of safety relief valves can be adequately detected if at least 5 of the 11 valves are bench tested every 24 months so that all valves are tested every 48 months."

2.0 EVALUATION

FitzPatrick has eleven safety/relief valves (SRVs) which are located on the main steam lines within the drywell between the reactor vessel and the inboard main steam isolation valves. Seven of these valves are automatic depressurization system (ADS) valves. Each SRV is equipped with two acoustical detectors or accelerometers (primary indication), one of which is maintained in service. Each valve is also equipped with a backup thermocouple detector (secondary indication). The acoustic detectors monitor noise level

and provide control room alarm upon indication of an open SRV. The thermocouples provide continuous monitoring and recording of SRV discharge temperature to detect valve leakage.

2.1 ADS Surveillance Testing

Technical Specification 4.5.D.1 requires a once per cycle surveillance testing of the ADS. This surveillance test includes a simulated automatic actuation to open all pilot valves and a simulated automatic actuation which is inhibited by the override switches. The existing on-line testing provides adequate assurance of valve operability. The safety relief valve monitor instrument check, performed on a monthly basis, demonstrates the ability of SRV tailpipe acoustic system to detect leaking or partially open SRVs by recording noise levels from each SRV accelerometer.

Another consideration for surveillance interval extension is the past performance of the SRVs. Occurrence reports were reviewed which indicated that SRV setpoint drift is a concern. However, the licensee requested, by letter dated December 20, 1989, and supplemented by letter dated March 2, 1994, to incorporate a single nominal SRV setting of 1110 psig and a 3% setpoint tolerance for the SRVs. This request was approved in Amendment No. 217, dated September 28, 1994. The analyses performed to support this TS change demonstrated the adequacy of plant piping and containment structures for the SRV setpoint and tolerance change. In addition, the licensee has reconfirmed a General Electric Company analysis, dated November 1991, which supported the SRV setpoint tolerance change. It was determined that SRV drift with a 3% tolerance would be acceptable for (i.e., bounded by) a 24- to 30-month interval.

The licensee has also assessed potential SRV drift over a 60-month period since proposed Specification 4.6.E.1 allows a maximum test frequency of 60 months. It was concluded that SRV drift, as measured by surveillance data, is not dependent on the time between surveillance intervals. Therefore, there is reasonable assurance that the SRVs will not drift excessively during a potential maximum 60-month period between surveillances. It should be noted that the NRC staff has approved similar SRV surveillance test extensions for both Limerick Units 1 and 2 and Peach Bottom Units 2 and 3, as discussed in the NRC's safety evaluations for these plants dated August 20, 1992, and August 19, 1992, respectively.

The licensee is also participating in the Boiling Water Reactors Owners' Group (BWROG) SRV setpoint drift program. Recommendations from this program are evaluated in reference to the FitzPatrick plant SRVs.

The licensee stated that the ADS surveillance frequency can be safely extended because past performance indicates, aside from the setpoint drift concerns, the SRVs have been mechanically reliable; and, on-line testing provides assurance of valve operability with leaking or partially open SRVs being detected by the acoustic monitor and associated alarms. The NRC staff agrees and the proposed TS changes is therefore acceptable.

2.2 SRV Position Indicator Functional Test, Item 19, Table 4.2-8.

Table 4.2-8 requires a once per operating cycle functional test of the safety/relief valve position indicators (accelerometers and thermocouples). The performance of this test requires drywell access. Therefore, it cannot be performed during power operation.

Test results from the previous four refueling outages were reviewed to determine system reliability. No failures of the functional test criteria were noted. Out of 88 accelerometer readings, there were only three instances where the accelerometers failed to register. One of the failures resulted from a maintenance activity, not equipment degradation. In none of the cases were both the primary (in service) and backup (out of service) accelerometers simultaneously inoperable. Therefore, operators have had the ability to detect SRV leakage or actuation. In addition, on-line monthly instrument checks, as required by TS Table 4.2-8, demonstrate the operability of the safety relief valve tailpipe acoustic system by measuring noise levels from each SRV.

The licensee stated that this functional test can be safely extended because the monthly surveillance of the SRV accelerometers and thermocouples ensures their operability; and, a review of past performance of the acoustic monitors and thermocouples indicates they have been reliable. The NRC staff agrees and the proposed TS changes is therefore acceptable.

2.3 Bench testing, inspection/disassembly, and manual operation testing of SRVs, Specifications 4.6.E.1, 4.6.E.2, and 4.6.E.4.

Specification 4.6.E.1 requires that 5 out of 11 SRVs be bench checked or replaced with bench checked valves once per operating cycle. It also requires that all valves be tested every two operating cycles. The purpose of this requirement is to verify that SRV setpoints are within a specified tolerance.

Specification 4.6.E.2 requires the disassembly and inspection of at least one SRV once each operating cycle. The purpose of this requirement is to identify early degradation of the valves. Surveillances 4.6.E.1 and 4.6.E.2 must be performed when the reactor is in cold shutdown with the drywell de-inerted.

Specification 4.6.E.4 requires the manual operation of each SRV to observe a greater than or equal to 10 percent closure of the turbine bypass valves. The manual operation of the valves also satisfies inservice testing exercise requirements.

In the specifications above, the monthly testing requirements serve to provide a redundant and early means of detecting SRV degradation. As discussed previously, the licensee has received approval, in Amendment No. 217, for SRV TS changes to increase the maximum permissible setpoint drift to $\pm 3\%$. The supporting evaluations for this amendment indicate that SRV setpoint drift is not expected to be a concern over a 24- to 30-month interval since the historical drift data was typically within the new tolerances of $\pm 3\%$.

It should be noted that the most recent occurrence of setpoint drift, as reported in Licensee Event Report (LER) 95-001, "Reactor Safety Relief Valve Setpoint Drift," dated February 4, 1995, was reported under the provision of 10 CFR 50.73(a)(2)(i)(B) as an operation of the plant in a condition prohibited by the TSs that were in effect during the past cycle (i.e., previous to Amendment No. 217). The remote actuation of the ADS functions would not have been effected by this event. The new specifications allow continuous operation with two SRVs inoperable and a setpoint drift of $\pm 3\%$. An analysis by the licensee has shown that continuous operation of the plant would be acceptable with two SRVs inoperable and nine SRVs actuating at 1195 psig. The analysis further confirmed that the setpoint drift of nine SRVs to the 1195 psig limit would not adversely affect the following:

- HPCI system
- RCIC system
- Primary Containment Integrity system
- Fuel Thermal Limits
- ECCS/LOCA performance

The licensee continues to subject all SRVs, rather than 5 out of 11 as specified in the TSs, to testing, refurbishment, and recertification once each operating cycle. The NRC staff agrees and the proposed TS changes are therefore acceptable.

2.4 Administrative Changes

The Bases change to TS Section 4.6, consistent with GL 91-04, clarifies the bench testing frequency of 24-months and does not impact any SRV safety limits.

The asterisk and footnote for these specifications on TS pages 142a and 143 are deleted as an administrative change since the conditions have expired. This one-time extension of SRV bench checking requirements is no longer valid. No safety limits are impacted by this administrative change.

The NRC staff agrees with the licensee's safety evaluation and the proposed TS changes are therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types,

of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 24916). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. E. Carpenter

Date: October 13, 1995

October 13, 1995

Mr. William J. Cahill, Jr.
Chief Nuclear Officer
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT (TAC NO. M92039)

Dear Mr. Cahill:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

C. E. Carpenter, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 229 to DPR-59
2. Safety Evaluation

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