



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931

April 4, 2001

EA-01-096

Southern Nuclear Operating Company, Inc.
ATTN.: Mr. D. N. Morey
Vice President
P. O. Box 1295
Birmingham, AL 35201-1295

**SUBJECT: WITHDRAWAL OF NON-CITED VIOLATION 50-348,364/00-01-01, FAILURE
TO IDENTIFY AN UNREVIEWED SAFETY QUESTION**

Dear Mr. Morey:

On December 18, 1997, the Farley Updated Final Safety Analysis Report (UFSAR) was changed to allow for higher sustained Reactor Vessel Support (RVS) concrete temperatures. The Farley technical staff determined this UFSAR change was not an Unreviewed Safety Question (USQ) based on interpretation of the American Concrete Institute (ACI) code. 10 CFR 50.59 requires that any change to the plant, including the UFSAR, that results in a USQ determination must be reviewed and approved by the NRC. By memorandum dated December 8, 1998, NRC Region II issued Task Interface Agreement (TIA) 98-11, Farley's Interpretation of ACI Code for Reactor Vessel Support Concrete Temperatures, requesting the NRC technical staff evaluate the UFSAR change and determine if a USQ existed. As documented in the response to TIA 98-11, dated January 24, 2000, the NRC technical staff concluded the UFSAR change was a USQ. Based on the NRC technical staff's conclusion, the subject Non-cited Violation (NCV) was issued in NRC Inspection Report Nos. 50-348/00-01 and 50-364/00-01 dated March 23, 2000.

By letter dated May 31, 2000, you denied the NCV and provided additional supporting information. By memorandum dated June 23, 2000, Region II issued TIA 2000-10, Reevaluation of TIA 98-11 Regarding Reactor Vessel Support Concrete Temperature At Farley Nuclear Plant, requesting the NRC technical staff review the additional supporting information and either reconfirm or modify their response to TIA 98-11. As a result of their review, the NRC technical staff identified two technical concerns; differential settling of the reactor vessel supports and reactor cavity liner bulging. On September 19, 2000, discussions between the NRC technical staff and Southern Nuclear Operating Company (SNC) personnel were held to resolve these two technical concerns. By letter dated November 10, 2000, you stated that differential settling of the reactor vessel supports would be detected during routine activities such as reactor head or reactor vessel internals removal. To detect reactor cavity liner bulging, you committed to include monitoring of RVS concrete structural condition in the Farley Maintenance Rule program.

As documented in the response to TIA 2000-10, dated March 6, 2001, the NRC technical staff believes that SNC is adequately monitoring the RVS concrete condition and will be able to detect any malfunction resulting from RVS concrete degradation; therefore, no USQ exists. Since this UFSAR change does not involve a USQ, NCV 50-348,364/00-01-01 is formally withdrawn. Accordingly, we will adjust our records to reflect that no violation of regulatory requirements occurred. This additional information was not available to our inspectors during the inspection.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter will be publicly available in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html>.

We appreciate your cooperation in this matter. If you have any questions, please call me at (404)562-4520.

Sincerely,

Stephen J. Cahill, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Docket Nos. 50-348, 50-364
License Nos. NPF-2, NPF-8

cc: (See page 3)

SNC

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