



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 21, 1995

Mr. William J. Cahill, Jr.  
Chief Nuclear Officer  
Power Authority of the State of New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER  
PLANT (TAC NO. M92397)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 230 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated May 12, 1995.

The amendment modifies the TSs to extend the surveillance test intervals for the emergency service water system to support 24-month operating cycles. Surveillance test interval extensions are denoted as being performed "every 24 months" or "at least once per 24 months" consistent with the guidance provided in Generic Letter (GL) 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate 24-Month Fuel Cycle," dated April 2, 1991. The NRC staff has determined that the proposed TS changes to the emergency service water system surveillance tests can be safely extended to accommodate a 24-month operating cycle, in accordance with GL 91-04, and are therefore acceptable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

C. E. Carpenter, Jr., Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 230 to DPR-59  
2. Safety Evaluation

cc w/encls: See next page

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Power Authority of the State  
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James A. FitzPatrick Nuclear  
Power Plant

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DATED: December 21, 1995

AMENDMENT NO. 230 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

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PDI-1 Reading

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C. McCracken

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S. Little

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OGC

G. Hill (2), T-5 C3

C. Grimes, 11/E/22

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cc: Plant Service list

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 230  
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated May 12, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 230, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Ledyard B. Marsh, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 21, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 230

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

240

241

Insert Pages

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**JAFNPP**

**3.11 (cont'd)**

**D. Emergency Service Water System**

1. To ensure adequate equipment and area cooling, both ESW systems shall be operable when the requirements of specification 3.5.A and 3.5.B must be satisfied, except as specified below in specification 3.11.D.2.

**4.11 (Cont'd)**

**D. Emergency Service Water System**

1. Surveillance of the ESW system shall be performed as follows:

<u>Item</u>	<u>Frequency</u>
a. Simulated Automatic Actuation Test	Once every 24 months
b. Flow Rate Test - Each ESW pump shall deliver at least 1500 gpm to its respective loop. The pump total developed head shall be greater than or equal to the corresponding point on the pump curve, reduced by a maximum of 7%, for the measured flow.	Once/3 months
c. Pump Operability	Once/month
d. Motor Operated Valves	Once/month

**JAFNPP**

**3.11 (cont'd)**

**4.11 (cont'd)**

2. From and after the time that one Emergency Service Water System is made or found to be inoperable for any reason continued reactor operation is permissible for a period not to exceed 7 days, provided that:
  - the operable Emergency Diesel Generator System is demonstrated to be operable immediately and daily thereafter; and,
  - all Emergency Diesel Generator System emergency loads are verified operable immediately and daily thereafter.
3. If specification 3.11.D.2 cannot be met the reactor shall be placed in the cold condition within 24 hours.

- |    |                                    |                      |
|----|------------------------------------|----------------------|
| e. | ESW instrumentation check          | Once/day             |
|    | ESW instrument channel calibration | Once/3 months        |
| f. | Logic System Functional Test       | Once every 24 months |

2. ESW will not be supplied to RBCLC system during testing.
3. Not Used



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 230 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated May 12, 1995, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant (FitzPatrick) Technical Specifications (TSs). The requested changes would revise TS Section 4.11.D, Emergency Service Water System, to support the extended operating cycle. The proposed change in test frequency is to every 24 months. These changes are necessary to avoid an extended mid-cycle outage.

The NRC staff has previously reviewed requests for individual plants to modify surveillance intervals to be compatible with a 24-month fuel cycle. Therefore, in a letter dated April 2, 1991, the staff issued Generic Letter (GL) 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate 24-Month Fuel Cycle," to provide licensees with generic guidance on preparing such license amendments.

2.0 EVALUATION

The service water system consists of three subsystems: the emergency service water system (ESW), the normal service water system, and the residual heat removal (RHR) service water system. The specific surveillances discussed in this safety evaluation are associated with the ESW system. The other two systems do not require surveillance test extensions to 24 months since there are no TS required once per cycle surveillance tests associated with these systems. The monthly operability test of the RHR service water pumps and associated motor-operated valves (MOVs) in Specification 4.5.B.1.c.1 is not impacted by the amendment request.

The ESW system is a safety-related system providing heat removal for the emergency core cooling system (ECCS) components and other equipment essential to safe reactor shutdown. The system consists of two independent supply loops, each supplied by a 100% capacity, motor driven, vertical turbine pump. Each pump takes suction from a separate location in the screenwell and discharges through independent strainers into separate supply headers. The system also includes five MOVs (two pump discharges, two bypass valves, and

one cross-connect). Each train of the ESW system is required to supply raw water cooling to one train of the following safety related loads:

- Emergency Diesel Generator Jacket Water Heat Exchangers,
- Electric Bay Unit Coolers,
- Cable Tunnel/Switchgear Room Coolers,
- Control Room and Relay Room Air Handling Units, and
- Crescent Area Unit Coolers

Normally, the ESW system is maintained in standby condition and operates automatically in response to an indicated loss of reactor building closed loop cooling water system or upon start of one or more emergency diesel generators.

The longer cycle length requires an extension to the ESW simulated automatic actuation test and logic system functional test. This surveillance test demonstrates that the reactor building closed loop cooling (RBCLC) pump discharge header pressure switches and ESW Lockout Matrix relays will cause ESW pumps to start and RBCLC and ESW MOVs to reposition to an ESW injection lineup. Ten surveillance test results were reviewed from 1987 to 1993. Two problems were noted with contacts on an ESW lockout relay failing to operate. The relay was replaced with a new one and the surveillance frequency was increased to quarterly. After seven consecutive tests were performed satisfactorily, the testing frequency was returned to once per operating cycle.

Extension of this test to 24 months will not adversely affect system performance or reliability because the relays used by this system have proven to be reliable as documented by numerous tests; and, all pumps and valves operated by this system are frequently tested with the plant on-line. Flow rate tests are performed quarterly, pump and MOV operability are checked monthly, instrumentation is checked daily and is calibrated quarterly.

In addition to the surveillance interval extensions for TSs 4.11.D.1.a and 4.11.D.1.f, Specification 4.11.D.1.e has been administratively rearranged to better clarify the requirements for ESW instrumentation check and instrument channel calibration. The "once/3 months" frequency, which appears twice in this Specification, has been aligned with the proper (single) surveillance test. No new or different tests or surveillance intervals are proposed by this administrative clarification.

Based on the discussion above, the ESW surveillance tests can be safely extended to accommodate a 24-month operating cycle.

The assumptions in the Fitzpatrick licensing basis are not invalidated by performing the ESW surveillances at the bounding interval limits (30 months) to accommodate the 24-month operating cycle.

The NRC staff has reviewed the proposed changes to TS Section 4.11.D and has concluded that they are in accordance with GL 91-04 and are therefore acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 47623). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. E. Carpenter, Jr.

Date: December 21, 1995



December 21, 1995

Mr. William J. Cahill  
Chief Nuclear Officer  
Power Authority of the State of New York  
123 Main Street  
White Plains, NY 10601

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

C. E. Carpenter, Jr., Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

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2. Safety Evaluation

cc w/encls: See next page

Distribution: See attached sheet

\*See previous concurrence

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