



50-333

UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 28, 1994

Mr. William J. Cahill, Jr.
Executive Vice President - Nuclear
Generation
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M90050)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 221 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated August 4, 1994, as supplemented November 10, 1994.

The amendment revises primary containment atmosphere monitoring and drywell to torus differential pressure requirements. Specifically, TS 3.7.A.6. has been revised to adopt primary containment inerting/deinerting requirements that are consistent with NUREG-1433, "Standard Technical Specifications - General Electric Plants, BWR/4." TSs 4.7.A.6.a. and 4.7.A.7.a. have been revised to provide frequencies for the verification of primary containment oxygen concentration and pressure differential between the drywell and torus. TSs 3.7.A.7.a.(1), 3.7.A.7.a.(3), and 3.7.A.8. have been revised to provide requirements for establishing and maintaining differential pressure between the drywell and torus that are consistent with NUREG-1433. TS 3.7.A.9. has been deleted and related requirements have been incorporated into Notes for TS Table 3.2-8. Several administrative changes to Tables 3.2-8 and 4.2-8 have also been made to improve the overall quality of the TSs.

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W. Cahill

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December 28, 1994

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 221 to DPR-59
2. Safety Evaluation

cc w/encls: See next page

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DATE	12/8/94		12/9/94		12/16/94		12/2/94		12/14/94	<i>for</i>

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W. Cahill

-2-

December 28, 1994

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



for Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 221 to DPR-59
2. Safety Evaluation

cc w/encls: See next page

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James A. FitzPatrick Nuclear
Power Plant

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DATED: December 28, 1994

AMENDMENT NO. 221 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

J. Zwolinski, 14/H/3

M. J. Case

C. Vogan

N. Conicella

OGC

D. Hagan, T-4 A43

G. Hill (2), T-5 C3

C. Grimes, 011/E/22

ACRS (4)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

R. Barrett, 08/H/7

cc: Plant Service list

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 221
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated August 4, 1994, as supplemented November 10, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 221, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



for Michael J. Case, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1994

primary containment oxygen concentration and of proper differential pressure between the drywell and torus at intervals that are the same as or more frequent than specified in NUREG-1433. The proposed changes are, therefore, acceptable.

2.3 Changes to TSs 3.7.A.7.a.(1), 3.7.A.7.a.(3), and 3.7.A.8.

TS 3.7.A.7.a.(1) currently requires that the required differential pressure between the drywell and torus be established within 24 hours of placing the reactor in the run mode during startup. TS 3.7.A.7.a.(1) also currently allows the differential pressure to be reduced to less than the required value 24 hours prior to a scheduled shutdown. The licensee has proposed that TS 3.7.A.7.a.(1) be revised to require the drywell to torus differential pressure to be established within 24 hours of exceeding 15 percent thermal power during startup. The proposed changes would allow the differential pressure to be reduced below the required value up to 24 hours prior to reducing thermal power to less than 15 percent of rated before a plant shutdown. The proposed changes are consistent with Section 3.6.2.5, "Drywell to Suppression Chamber Differential Pressure," of NUREG-1433.

TS 3.7.A.7.a.(3) currently requires that if drywell to torus differential pressure cannot be maintained as required and cannot be restored within the subsequent 6 hours, the reactor must be placed in hot shutdown in 6 hours and in cold shutdown in the following 18 hours. The licensee has proposed changes to TS 3.7.A.7.a.(3) that would require differential pressure to be restored within 8 hours or reactor power to be reduced to less than 15 percent of rated within the next 12 hours if drywell to torus differential pressure cannot be maintained as required. The proposed changes to TS 3.7.A.7.a.(3) are consistent with Section 3.6.2.5 of NUREG-1433.

The licensee has also proposed that TS 3.7.A.8. be revised to reflect that the revised TS 3.7.A.6. does not require shutdown to the cold condition within 24 hours.

The staff has determined that the proposed changes to TSs 3.7.A.7.a.(1), 3.7.A.7.a.(3), and 3.7.A.8. are consistent with the guidance in NUREG-1433 and will help facilitate maintenance and inspection activities in the drywell during startups as previously discussed in Section 2.1. The proposed changes are, therefore, acceptable.

2.4 Changes to Notes for Table 3.2-8

Note F for Table 3.2-8 currently refers to TS 3.7.A.9. if minimum requirements for hydrogen/oxygen monitoring instrumentation are not met. The licensee has proposed that the reference to TS 3.7.A.9. be eliminated and that the associated action statement be relocated to Note F. The current action statement requirement would also be modified such that if recorder 27PCR-101A or B is inoperable, a daily monitoring and logging of the appropriate

parameter on associated indicator panel 27PCX-101A or B would be acceptable in lieu of taking grab samples. The monitoring would be performed on 27PCX-101A or B using Regulatory Guide 1.97 qualified analyzers. The action statement would also be modified to require that the plant be placed in a mode in which monitoring is not required (i.e., hot shutdown) if at least one channel is not made operable within a 30-day period. The licensee has proposed that the required completion time for this action be 12 hours, consistent with NUREG-1433.

The licensee has also proposed that a new Note K be added to Table 3.2-8 that would specify when the primary containment atmosphere must be monitored for hydrogen and oxygen. TS 3.7.A.9. would be deleted since the requirements in this TS would be contained in Notes F and K for Table 3.2-8.

The staff has determined that the proposed change to allow the daily monitoring and logging of data from indicator panel 27PCX-101A or B will provide data equivalent to that obtained from grab samples. The proposed action to be taken if the number of operable primary containment hydrogen/oxygen instrument channels is less than the required minimum and the specified compensatory measures are not taken is consistent with NUREG-1433. The staff has also determined that the deletion of TS 3.7.A.9. and incorporation of related requirements into Notes F and K for Table 3.2-8 is administrative in nature. The proposed changes to the Notes for Table 3.2-8 and the deletion of TS 3.7.A.9. are, therefore, acceptable.

2.5 Administrative Changes

The licensee has proposed several administrative changes to improve the overall quality of the TSs. The specific changes are discussed below.

TS 4.2.H. would be modified by adding the words "functional test" to the list of tests performed and adding the words "as applicable" at the end of the sentence. TS 4.7.A.9. would be deleted since the changes to TS 4.2.H. would effectively incorporate the requirements of TS 4.7.A.9. into TS 4.2.H.

Items 11 on Tables 3.2-8 and 4.2-8 would be revised by changing the references to "drywell" to "primary containment."

The staff has confirmed that these proposed changes to the TSs are administrative in nature and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 45032). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Menning

Date: December 28, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 221

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

54
77b
77d
86
180
180a
181

Insert Pages

54
77b
77d
86
180
180a
181

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3.2 (cont'd)

E. Drywell Leak Detection

The limiting conditions of operation for the instrumentation that monitors drywell leak detection are given in Table 3.2-5.

F. (Deleted)

G. Recirculation Pump Trip

The limiting conditions for operation for the instrumentation that trip(s) the recirculation pumps as a means of limiting the consequences of a failure to scram during an anticipated transient are given in Table 3.2-7.

H. Accident Monitoring Instrumentation

The limiting conditions for operation of the instrumentation that provides accident monitoring are given in Table 3.2-8.

I. 4kv Emergency Bus Undervoltage Trip

The limiting conditions for operation for the instrumentation that prevents damage to electrical equipment or circuits as a result of either a degraded or loss-of-voltage condition on the emergency electrical buses are given in Table 3.2-2.

4.2 (cont'd)

E. Drywell Leak Detection

Instrumentation shall be calibrated and checked as indicated in Table 4.2-5.

F. (Deleted)

G. Recirculation Pump Trip

Instrumentation shall be functionally tested and calibrated as indicated in Table 4.2-7.

System logic shall be functionally tested as indicated in Table 4.2-7.

H. Accident Monitoring Instrumentation

Instrumentation shall be demonstrated operable by performance of a channel check, channel calibration and functional test as indicated in Table 4.2-8, as applicable.

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TABLE 3.2-8 (Cont'd)
ACCIDENT MONITORING INSTRUMENTATION

NOTES FOR TABLE 3.2-8

- A. With the number of operable channels less than the required minimum, either restore the inoperable channels to operable status within 30 days, or be in a cold condition within the next 24 hours.
- B. With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, initiate an alternate method of monitoring the appropriate parameter(s) within 72 hours and: (1) either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or (2) prepare and submit a Special Report to the Commission within 14 days following the event outlining the cause of the inoperability, the action taken, and the plans and schedule for restoring the system to OPERABLE status.
- C. Each Safety/Relief Valve is equipped with two acoustical detectors, one of which is in service. Each SRV also has a backup thermocouple detector. In the event that a thermocouple is inoperable, SRV performance shall be monitored daily with the associated in service acoustical detector.
- D. From and after the date that both of the acoustical detectors are inoperable, continued operation is permissible until the next outage in which a primary containment entry is made provided that the thermocouple is operable. Both acoustical detectors shall be made operable prior to restart.
- E. In the event that both primary (acoustical detectors) and secondary (thermocouple) indications of this parameter for any one valve are disabled and neither indication can be restored in forty-eight (48) hours, the reactor shall be in a Hot Shutdown condition within twelve (12) hours and in a Cold Shutdown within the next twenty-four (24) hours.
- F. With the number of operable channels less than the required minimum, continued reactor operation is permissible for the following 30 days provided at least once each 24 hours, either the appropriate parameter(s) is monitored and logged using 27PCX-101A, B, or an appropriate grab sample is obtained and analyzed. If this condition can not be met, be in the Hot Shutdown mode within the next 12 hours.
- G. This parameter and associated instrumentation are not part of post-accident monitoring.
- H. This instrument shall be operable in the Run, Startup/Hot Standby, and Hot Shutdown modes.
- J. This instrument shall be operable in the Run and Startup/Hot Standby modes.
- K. Primary containment atmosphere shall be continuously monitored for hydrogen and oxygen when in the Run and Startup/Hot Standby modes, except when the Post-Accident Sampling System (PASS) is to be operated. When the PASS is to be operated, the containment atmosphere monitoring systems may be isolated for a period not to exceed 3 hours in a 24-hour period.

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TABLE 3.2-8

ACCIDENT MONITORING INSTRUMENTATION

Instrument	No. of Channels Provided by Design	Minimum No. of Operable Channels Required	Mode in Which Instrument Must be Operable	Action
8. Torus Water Level (wide range) (23LI-202A or 23LR-202A/203A) (23LI-202B or 23LR-202B/203B)	2	1	Note J	Note A
9. Torus Bulk Water Temperature (16-1TI-131A or 16-1TR-131A) (16-1TI-131B or 16-1TR-131B)	2	1	Note J	Note A
10. Torus Pressure (27PR-101A) (27PR-101B1)	2	1	Note J	Note A
11. Primary Containment Hydrogen/Oxygen Concentration (27PCR-101A) (27PCR-101B)	2	1	Note J, K	Note F
12. Reactor Vessel Pressure (06PI-61A or 06PR-61A) (06PI-61B or 06PR-61B)	2	1	Note J	Note A
13. Reactor Water Level (fuel zone) (02-3LI-091) (02-3LR-098)	2	1	Note J	Note A
14. Reactor Water Level (wide range) (02-3LI-85A) (02-3LR-85B)	2	1	Note J	Note A

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TABLE 4.2-8

**MINIMUM TEST AND CALIBRATION FREQUENCY FOR
ACCIDENT MONITORING INSTRUMENTATION**

Instrument	Instrument Functional Test	Calibration Frequency	Instrument Check
1. Stack High Range Effluent Monitor	Once/Operating Cycle	Once/Operating Cycle	Once/day
2. Turbine Building Vent High Range Effluent Monitor	Once/Operating Cycle	Once/Operating Cycle	Once/day
3. Radwaste Building Vent High Range Effluent Monitor	Once/Operating Cycle	Once/Operating Cycle	Once/day
4. Containment High Range Radiation Monitor	Once/Operating Cycle	Once/Operating Cycle	Once/day
5. Drywell Pressure (narrow range)	N/A	Once/Operating Cycle	Once/day
6. Drywell Pressure (wide range)	N/A	Once/Operating Cycle	Once/day
7. Drywell Temperature	N/A	Once/Operating Cycle	Once/day
8. Torus Water Level (wide range)	N/A	Once/Operating Cycle	Once/day
9. Torus Bulk Water Temperature	N/A	Once/Operating Cycle	Once/day
10. Torus Pressure	N/A	Once/Operating Cycle	Once/day
11. Primary Containment Hydrogen/Oxygen Concentration Analyzer	N/A	Once/3 months	Once/day
12. Reactor Vessel Pressure	N/A	Once/Operating Cycle	Once/day
13. Reactor Water Level (fuel zone)	N/A	Once/Operating Cycle	Once/day
14. Reactor Water Level (wide range)	N/A	Once/Operating Cycle	Once/day

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3.7 (cont'd)

6. Oxygen Concentration

The primary containment oxygen concentration shall be maintained less than 4.0 volume percent while in the Run mode, except as specified in 3.7.A.6.a and 3.7.A.6.b below:

- a. Primary containment oxygen concentration shall be less than 4.0 volume percent within 24 hours of exceeding 15% of rated thermal power during startup.
- b. De-inerting may commence up to 24 hours prior to reducing thermal power to less than 15% of rated before a plant shutdown.
- c. If oxygen concentration is greater than or equal to 4.0 volume percent at any time while in the Run mode, except as specified in 3.7.A.6.a or 3.7.A.6.b above, restore oxygen concentration to less than 4.0 volume percent within 24 hours, otherwise reduce thermal power to less than or equal to 15% of rated within the next 8 hours.

7. Drywell-Torus Differential Pressure

- a. Differential pressure between the drywell and torus shall be maintained at equal to or greater than 1.7 psid except as specified in (1) and (2) below:

4.7 (cont'd)

6. Oxygen Concentration

- a. The primary containment oxygen concentration shall be verified to be within limits once each week. Instrument surveillances shall be performed as specified in Table 4.2-8.

7. Drywell-Torus Differential Pressure

- a. The pressure differential between the drywell and torus shall be verified to be within limits once each shift. Instrument surveillances shall be performed as specified in Table 4.2-8.

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3.7 (Cont'd)

- (1) The drywell to torus differential pressure shall be established within 24 hours of exceeding 15% rated thermal power during startup. The differential pressure may be reduced to less than the limit up to 24 hours prior to reducing thermal power to less than 15% of rated before a plant shutdown.
 - (2) The differential pressure may be decreased to less than 1.7 psid for a maximum of four (4) hours during required operability testing of the HPCI, RCIC, and Suppression Chamber - Drywell Vacuum Breaker System.
 - (3) If 3.7.A.7.a above cannot be met, restore the differential pressure to within limits within eight hours or reduce thermal power to less than 15% of rated within the next 12 hours.
8. If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met the reactor shall be in the cold condition within 24 hours.

4.7 (Cont'd)

8. Not applicable.

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3.7 (cont'd)

4.7 (cont'd)

B. Standby Gas Treatment System

1. Except as specified in 3.7.B.2 below both circuits of the Standby Gas Treatment System shall be operable at all times when secondary containment integrity is required.

B. Standby Gas Treatment System

1. Standby Gas Treatment System surveillance shall be performed as indicated below:
 - a. At least once per operating cycle, it shall be demonstrated that:
 - (1) Pressure drop across the combined high-efficiency and charcoal filters is less than 5.7 in. of water at 6,000 scfm, and
 - (2) Each 39kW heater shall dissipate greater than 29kW of electric power as calculated by the following expression:

$$P = \sqrt{3}EI$$

where:

P= Dissipated Electrical Power;

E= Measured line-to-line voltage in volts (RMS);

I= Average measured phase current in amperes (RMS).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 221 TO FACILITY OPERATING LICENSE NO. DPR-59
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated August 4, 1994, and supplemented November 10, 1994, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise primary containment atmosphere monitoring and drywell to torus differential pressure requirements. Specifically, TS 3.7.A.6. would be revised to adopt primary containment inerting/deinerting requirements that are consistent with NUREG-1433, "Standard Technical Specifications - General Electric Plants, BWR/4." TSs 4.7.A.6.a. and 4.7.A.7.a. would be revised to provide frequencies for the verification of primary containment oxygen concentration and pressure differential between the drywell and torus. TSs 3.7.A.7.a.(1), 3.7.A.7.a.(3), and 3.7.A.8. would be revised to provide requirements for establishing and maintaining differential pressure between the drywell and torus that are consistent with NUREG-1433. TS 3.7.A.9. would be deleted and the requirements of this TS would be incorporated into Notes for TS Table 3.2-8. Several administrative changes to Tables 3.2-8 and 4.2-8 would also be made to improve the overall quality of the TSs. The November 10, 1994, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

During normal operation and following a design-basis accident (i.e., loss-of-coolant accident (LOCA)) the primary containment must be inerted to ensure that hydrogen combustion cannot occur. Inerting is achieved by purging the primary containment with nitrogen until the oxygen concentration is less than 4 percent. When the primary containment is inerted, operators cannot access the containment to perform required surveillances and leak inspections unless they wear self contained breathing apparatus. Nitrogen is also used to maintain the drywell to torus differential pressure at greater than or equal to 1.7 psig. This differential pressure is necessary to reduce water slug forces on the torus and jet forces on the downcomer piping that could result during a LOCA.

2.0 EVALUATION

2.1 Changes to TS 3.7.A.6.

TS 3.7.A.6. currently requires that the containment atmosphere oxygen concentration be reduced to less than 4 percent within 24 hours after placing

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the reactor in the run mode during plant startups. TS 3.7.A.6. also currently allows deinerting to commence 24 hours prior to a shutdown. The licensee has proposed that TS 3.7.A.6. be revised to require the containment oxygen concentration to be less than 4 percent within 24 hours of exceeding 15 percent of rated thermal power during startups and to allow deinerting to commence up to 24 hours prior to reducing thermal power to less than 15 percent of rated thermal power prior to a plant shutdown. TS 3.7.A.b would also state that if oxygen concentration is greater than or equal to 4 percent while in the run mode, restore oxygen concentration to less than 4 percent within 24 hours, otherwise reduce thermal power to less than or equal to 15 percent within the next 8 hours. The proposed changes to TS 3.7.A.6. are consistent with Section 3.6.3.3, "Primary Containment Oxygen Concentration," of NUREG-1433.

The licensee requested the changes to TS 3.7.A.6. to increase overall plant reliability and capacity factor. The licensee has stated that the requested changes will allow operators to place the reactor mode switch in the run position sooner during startups. This will reduce the probability of spurious startup neutron monitoring instrumentation scrams. The licensee has also stated that the proposed changes will allow additional time for plant personnel to perform work/inspections in the drywell such as main steam isolation valve (MSIV) testing, MSIV limit switch adjustments, motor-operated valve testing, and leak inspections, before the containment is inerted.

The NRC staff has determined that the proposed changes to TS 3.7.A.6. are consistent with NUREG-1433 and with TS requirements that have been approved by the staff for other plants. The proposed changes are, therefore, acceptable.

2.2 Changes to TS 4.7.A.6.a. and 4.7.A.7.a.

TSs 4.7.A.6.a. and 4.7.A.7.a. currently require that the primary containment oxygen concentration and pressure differential between the drywell and torus, respectively, be monitored as specified in Table 4.2-8. However, these two TSs do not actually provide surveillance requirements for oxygen concentration and differential pressure since Table 4.2-8 only addresses surveillance requirements for related monitoring instrumentation. The licensee has, therefore, proposed that TSs 4.7.A.6.a. and 4.7.A.7.a. be revised to provide specific surveillance frequencies for the verification of primary containment oxygen concentration and of drywell to torus differential pressure. Verification of proper primary containment oxygen concentration and of proper differential pressure would be required once each week and once each shift, respectively.

The staff notes that Section 3.6.3.3.1 of NUREG-1433 requires the verification of proper primary containment oxygen concentration every 7 days, consistent with the licensee's proposal. Section 3.6.2.5.1 of NUREG-1433 requires the verification of proper differential pressure between the drywell and torus once every 12 hours. The licensee has proposed more frequent verification of proper differential pressure in view of its 8-hour operating shift schedule.

The staff has concluded that the proposed changes to TSs 4.7.A.6.a. and 4.7.A.7.a. will establish needed requirements of the verifications of proper