



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

January 18, 1995

*Docket  
File*

Mr. William J. Cahill, Jr.  
Executive Vice President - Nuclear  
Generation  
Power Authority of the State of  
New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: ADMINISTRATIVE ERROR IN ISSUANCE OF TECHNICAL SPECIFICATIONS  
AMENDMENT NOS. 220 AND 221 - JAMES A. FITZPATRICK NUCLEAR POWER  
PLANT (TAC NOS. M90430 AND M90050)

Dear Mr. Cahill:

Due to administrative errors, several pages transmitted to the Power Authority of the State of New York via the subject documents were incorrect. However, each error was minor and had no effect on the technical validity of the James A. FitzPatrick Nuclear Power Plant (FitzPatrick) Technical Specifications (TSs). Details of each error are as follows:

- Amendment No. 220, issued on December 20, 1994, revised, in part, page 253. However, the page transmitted with Amendment No. 220 inadvertently deleted part of a sentence, not related to the amendment, in TS Section 6.7(A). Enclosed is the correct page 253 that should have been issued with Amendment No. 220.
- Amendment No. 221, issued on December 22, 1994, revised, in part, pages 53 and 181. However, page 54 was transmitted with Amendment No. 221 rather than page 53. Page 54 had been repaginated into page 53 by Amendment No. 216, issued on August 31, 1994. The page 181 transmitted with Amendment No. 221 did not contain a revision bar in the left hand column. Enclosed are the correct pages 53 and 181, along with the correct instruction page that should have been issued with Amendment No. 221.

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*RJL*

*DF01*

W. Cahill

-2-

January 18, 1995

Please update the FitzPatrick TSs accordingly. If you have any questions, please contact me at (301) 415-1423.

Sincerely,

Original signed by

Clarence E. Carpenter, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure: Corrected TS Pages

cc w/encl: See next page

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W. Cahill

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January 18, 1995

Please update the FitzPatrick TSs accordingly. If you have any questions, please contact me at (301) 415-1423.

Sincerely,

*H. J. Condit For*

Clarence E. Carpenter, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure: Corrected TS Pages

cc w/encl: See next page

William J. Cahill, Jr.  
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Power Plant

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6.6 REPORTABLE EVENT ACTION

The following actions shall be taken for Reportable Events:

- (A) The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- (B) Each Reportable Event shall be reviewed by the PORC, and the results of this review shall be submitted to the Executive Vice President and Chief Nuclear Officer, the Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC.

6.7 SAFETY LIMIT VIOLATION

- (A) If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36, (c) (i).
- (B) An immediate report of each safety limit violation shall be made to the NRC by the Resident Manager. The Executive Vice President and Chief Nuclear Officer, the Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC will be notified within 24 hours.
- (C) The PORC shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluation of: (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility component systems or structures and (3) corrective action required to prevent recurrence. The Resident Manager shall forward this report to the Executive Vice President and Chief Nuclear Officer, the Vice President Regulatory Affairs and Special Projects, the Chairman of the SRC, and the NRC.

6.8 PROCEDURES

- (A) Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5 "Facility Administrative Policies and Procedures" of ANSI 18.7-1972 and Appendix A of Regulatory Guide 1.33, November 1972. In addition, procedures shall be established, implemented and maintained for the Fire Protection Program and other programs, as specified in Appendix B of the Radiological Effluent Technical Specifications, Section 7.2.
- (B) Those procedures affecting nuclear safety shall be reviewed by PORC and approved by the Resident Manager prior to implementation.
- (C) Temporary changes to nuclear related procedures may be made provided:
  1. The intent of the original procedure is not altered.

Amendment No. ~~50, 60, 65, 78, 93, 110~~, 220

ATTACHMENT TO LICENSE AMENDMENT NO. 221

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

53  
77b  
77d  
86  
180  
180a  
181

Insert Pages

53  
77b  
77d  
86  
180  
180a  
181

**JAFNPP**

3.2 (cont'd)

E. Drywell Leak Detection

The limiting conditions of operation for the instrumentation that monitors drywell leak detection are given in Table 3.2-5.

F. (Deleted)

G. Recirculation Pump Trip

The limiting conditions for operation for the instrumentation that trip(s) the recirculation pumps as a means of limiting the consequences of a failure to scram during an anticipated transient are given in Table 3.2-7.

H. Accident Monitoring Instrumentation

The limiting conditions for operation of the instrumentation that provides accident monitoring are given in Table 3.2-8.

I. 4kv Emergency Bus Undervoltage Trip

The limiting conditions for operation for the instrumentation that prevents damage to electrical equipment or circuits as a result of either a degraded or loss-of-voltage condition on the emergency electrical buses are given in Table 3.2-2.

4.2 (cont'd)

E. Drywell Leak Detection

Instrumentation shall be calibrated and checked as indicated in Table 4.2-5

F. (Deleted)

G. Recirculation Pump Trip

Instrumentation shall be functionally tested and calibrated as indicated in Table 4.2-7.

System logic shall be functionally tested as indicated in Table 4.2-7.

H. Accident Monitoring Instrumentation

Instrumentation shall be demonstrated operable by performance of a channel check, channel calibration and functional test as indicated in Table 4.2-8, as applicable.

I. Not Used

3.7 (cont'd)

4.7 (cont'd)

**B. Standby Gas Treatment System**

1. Except as specified in 3.7.B.2 below both circuits of the Standby Gas Treatment System shall be operable at all times when secondary containment integrity is required.

**B. Standby Gas Treatment System**

1. Standby Gas Treatment System surveillance shall be performed as indicated below:
  - a. At least once per operating cycle, it shall be demonstrated that:
    - (1) Pressure drop across the combined high-efficiency and charcoal filters is less than 5.7 in. of water at 6,000 scfm, and
    - (2) Each 39kW heater shall dissipate greater than 29kW of electric power as calculated by the following expression:

$$P = \sqrt{3}EI$$

where:

P= Dissipated Electrical Power;

E= Measured line-to-line voltage in volts (RMS);

I= Average measured phase current in amperes (RMS).