

Docket No. 50-333

April 13, 1994

Mr. William A. Josiger, Acting Executive
Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Josiger:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M88461)

The Commission has issued the enclosed Amendment No. 210 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 29, 1993.

The amendment revises TS 3.6.D.4 to eliminate an inconsistency between the operability requirements for the reactor coolant system (RCS) leakage detection and the specified requirements for monitoring RCS leakage. Additionally, the amendment revises TSs to make numerous editorial corrections which are administrative in nature.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:
Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 210 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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DATED: April 13, 1994

AMENDMENT NO. 210 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Vice President - Nuclear Generation
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Sincerely,

A handwritten signature in black ink that reads "Brian C. McCabe".

Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.210 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Power Authority of the State of New York

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Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 210
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated December 29, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

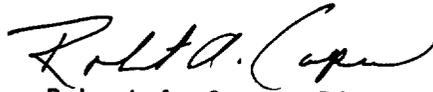
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 210, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 13, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 210

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

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141
141a
142
162a

Insert Pages

vi
141
141a
142
162a

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JAFNPP

3.6 (cont'd)

4. Except as specified in 3.6.C.3 above, the reactor coolant water shall not exceed the following limits with steaming rates greater than or equal to 100,000 lb/hr and during reactor shutdowns.

Conductivity	5 μ mho/cm
Chloride ion	0.5 ppm

5. If Specification 3.6.C cannot be met, the reactor shall be placed in a cold condition within 24 hours.

D. Coolant Leakage

1. Anytime irradiated fuel is in the reactor vessel and the reactor coolant temperature is above 212°F, the reactor coolant leakage into the primary containment shall be limited to:
 - a. 5 gpm unidentified leakage
 - b. 2 gpm increase in unidentified leakage within any 24 hour period. (This limitation shall apply only after a period of 24 hours at operating pressure.)
 - c. The total reactor coolant leakage into the primary containment shall not exceed 25 gpm.

4.6 (cont'd)

D. Coolant Leakage

1. Reactor coolant leakage rate inside the primary containment shall be monitored and recorded once every 4 hours utilizing the Primary Containment Sump Monitoring System (equipment drain sump monitoring and floor drain sump monitoring).

JAFNPP

3.6 (cont'd)

2. With reactor coolant system leakage greater than the limits specified in 3.6.D.1.a or 3.6.D.1.c, the leakage rate shall be reduced to within these limits within 4 hours or the reactor shall be in at least the hot standby condition within the following 12 hours and in cold condition within the next 24 hours.
3. With an increase in unidentified reactor coolant system leakage equal to or greater than the limit specified in 3.6.D.1.b, the source of the leakage shall be identified within 4 hours or the reactor shall be in at least hot standby condition within the next 12 hours and in cold condition within the following 24 hours.
4. The Primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring and Floor Drain Sump Monitoring) and the Continuous Atmosphere Monitoring System (Gaseous and Particulate) shall be operable when the reactor coolant leakage limits of Specification 3.6.D.1 are in effect.

4.6 (cont'd)

2. Not Used
3. Not Used
4. The Primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring and Floor Drain Sump Monitoring) instrumentation shall be calibrated and checked as specified in Surveillance Requirement 4.2.E. Continuous Atmosphere Monitoring System (Gaseous and Particulate) instrumentation shall be functionally tested and calibrated as specified in Table 4.6-2.

JAFNPP

3.6 (cont'd)

5. With the Primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring or Floor Drain Sump Monitoring) inoperable, restore the system to operable status within 24 hours or be in at least hot shutdown within the next 12 hours and in the cold condition within the following 24 hours.
6. With the Continuous Atmosphere Monitoring System (gaseous) or the Continuous Atmosphere Monitoring System (particulate) inoperable, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours. Otherwise be in at least hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours.

4.6 (cont'd)

JAFNPP

Table 4.6-2

Minimum Test and Calibration Frequency for Continuous Atmosphere Monitoring System

Inst. Channel	Inst. Functional Test	Calibration	Sensor Check
1. Air Particulate Analyzer	None	Once / 3 mos.	once / day
2. Gaseous Activity Analyzer	None	Once / 3 mos.	once / day



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 210 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated December 29, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise TS 3.6.D.4 to eliminate an inconsistency between the operability requirements for the reactor coolant system (RCS) leakage detection and the specified requirements for monitoring RCS leakage. In addition, the requested changes would also revise the TSs to make numerous editorial corrections which are administrative in nature.

2.0 EVALUATION

2.1 Reactor Coolant System Leakage Monitoring

The licensee has proposed to revise TS 3.6.D.4 to eliminate an inconsistency between the RCS leakage detection operability requirements and the requirements for monitoring leakage. TS 3.6.D.1 currently requires that RCS leakage be below specified limits when there is irradiated fuel in the reactor vessel and reactor coolant temperature is greater than 212 °F. TS 3.6.D.4 requires the leakage detection systems to be operable during power operation (any time the reactor is at a power level greater than 1 percent rated power, even at temperatures less than 212 °F).

The proposed change would require the leakage detection system to be operable consistent with the requirements for monitoring leakage. Although the change relaxes the requirement for the leakage detection system to be operable under certain power conditions, at temperatures of 212 °F and below, there are no TS requirements to monitor RCS leakage under these conditions. Therefore, the change would not decrease the margin of safety that is currently provided by the existing TSs. The change, however, would require the system to be operable whenever leakage monitoring is required. Currently, the leakage detection system is not required to be operable when the plant is shutdown but reactor coolant temperature is greater than or equal to 212 °F, although monitoring of RCS leakage is required under these conditions.

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The NRC staff finds that the proposed change would improve the consistency of the TSs while maintaining adequate RCS leakage monitoring capability. Based on the fact that adequate RCS leakage monitoring capability is maintained and there would be no decrease in the margin of safety currently provided the existing TSs, the NRC staff finds the proposed change to be acceptable.

2.2 Administrative Changes

The licensee has proposed miscellaneous typographical and editorial corrections to Appendix A TSs. The staff has reviewed these proposed changes and determined that they are acceptable since they do not involve any substantive changes to requirements.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 4945). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Replogle

Date: April 13, 1994