



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 24, 1993

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M85881)

The Commission has issued the enclosed Amendment No. 189 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated February 22, 1993.

The amendment revises TS 4.9.G.1 and associated Bases consistent with the guidance provided in Generic Letter 91-09, "Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System." TS 4.9.G.1 had previously required channel functional testing of the reactor protection system electrical protection assemblies at least once every 6 months. The revised TS 4.9.G.1 requires channel functional testing each time the plant is in cold shutdown for a period of more than 24 hours, unless performed in the previous 6 months. In addition, three minor editorial changes have been made to TS 4.9.G.2 to improve clarity.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Brian C. McCabe

Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 189 to DPR-59
2. Safety Evaluation

cc w/enclosures:

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Power Authority of the State of New York

James A. FitzPatrick Nuclear
Power Plant

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Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
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DATED: May 24, 1993

AMENDMENT NO. 189 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File
NRC & Local PDRs
PDI-1 Reading
S. Varga, 14/E/4
J. Calvo, 14/A/4
R. Capra
C. Vogan
J. Menning
B. McCabe
OGC-WF
D. Hagan, 3302 MNBB
G. Hill (2), P1-22
Wanda Jones, P-370
C. Grimes, 11/F/23
T. Dunning, 11/E/22
ACRS (10)
OPA
OC/LFMB
PD plant-specific file
C. Cowgill, Region I
B. Holian, 13/H/15

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 189
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated February 22, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 189, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 24, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 189

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

222c
226

Insert Pages

222c
226

JAFNPP

3.9 (cont'd)

3. From and after the time both power supplies are made or found inoperable the reactor shall be brought to cold condition within 24 hours.

G. REACTOR PROTECTION SYSTEM ELECTRICAL PROTECTION ASSEMBLIES

Two RPS electrical protection assemblies for each inservice RPS MG set and inservice alternate source shall be operable except as specified below:

1. With one RPS electrical protection assembly for an inservice RPS MG set or an inservice alternate power supply inoperable, restore the inoperable channel to operable status within 72 hours or remove the associated RPS MG set or alternate power supply from service.
2. With two RPS electrical protection assemblies for an inservice RPS MG set or an inservice alternate power supply inoperable, restore at least one to operable status within 30 minutes or remove the associated RPS MG set or alternate power supply from service.

4.9 (cont'd)

G. REACTOR PROTECTION SYSTEM ELECTRICAL PROTECTION ASSEMBLIES

The RPS electrical protection assemblies instrumentation shall be determined operable by:

1. Performing a channel functional test each time the plant is in cold shutdown for a period of more than 24 hours, unless performed in the previous 6 months.
2. At least once per operating cycle, demonstrating the operability of over-voltage, under-voltage and under-frequency protective instrumentation by performance of a channel calibration including simulated automatic actuation of the protective relays, tripping logic and output circuit breakers and verifying the following set points:

	<u>RPS MG SET SOURCE</u>
OVER-VOLTAGE	$\leq 132V$ ≤ 4 second Time Delay
UNDER-VOLTAGE	$\geq 108V$ ≤ 4 second Time Delay
UNDER-FREQUENCY	$\geq 57HZ$ ≤ 4 second Time Delay

4.9 BASES (con't)

D. Battery System

Measurements and electrical tests are conducted at specified intervals to provide indication of cell condition and to determine the discharge capability of the batteries. Performance and service tests are conducted in accordance with the recommendations of IEEE 450-1987.

E. LPCI MOV Independent Power Supply

Measurement and electrical tests are conducted at specified intervals to provide indication of cell condition, to determine the discharge capability of the battery. Performance and service tests are conducted in accordance with the recommendations of IEEE 450-1987.

F. Reactor Protection Power Supplies

Functional tests of the electrical protection assemblies are conducted at specified intervals utilizing a built-in test device and once per operating cycle by performing an instrument calibration which verifies operation within the limits of Section 4.9.G.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 189 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated February 22, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant, Technical Specifications (TS). The proposed changes modify the requirements for performing a channel functional test of electrical protective assemblies (EPAs) that are currently specified with a 6-month surveillance interval. Guidance on this proposed change was provided to all boiling-water reactor (BWR) licensees in Generic Letter (GL) 91-09, "Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System," that was issued on June 27, 1991. The licensee also proposed a conforming change to the Bases and three minor editorial changes to TS 4.9.G.2.

2.0 EVALUATION

The licensee has proposed to modify the 6-month surveillance interval for performing channel functional tests of EPAs as specified in TS 4.9.G.1 to state that they are to be performed "each time the plant is in cold shutdown for a period of more than 24 hours, unless performed in the previous 6 months." This change is consistent with the guidance provided in GL 91-09.

The Niagara Mohawk Power Corporation provided an analysis in a proposal submitted on December 15, 1988, that calculated the safety risks and benefits of this TS change. The NRC staff reviewed and concurred with the conclusion of this analysis that this TS change will produce a net safety benefit. Because the EPAs for all BWRs are primarily the same, the staff finds that this analysis applies generically to all BWRs. In addition, it is the staff's qualitative judgment that the proposed increase in the surveillance interval is not safety significant because of the diverse protection that exists, the number of failures that have to occur to have an adverse impact on safety, and the potential for detecting a degraded reactor protection system condition through on-line testing. Therefore, the staff finds that the licensee's proposed changes to TS 4.9.G.1 are acceptable.

The licensee has proposed a change to the TS Bases that would reflect the change to TS 4.9.G.1. The staff has no objections to the proposed Bases change.

The licensee has proposed three minor editorial changes to TS 4.9.G.2. Specifically, the licensee has proposed that in three locations "s" be replaced with "second." The staff finds these changes to be acceptable since they improve the clarity of the related requirements.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16870). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:
Thomas G. Dunning
John E. Menning

Date: May 24, 1993

May 24, 1993

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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Sincerely,
Original signed by:
Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 189 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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NAME	CVogan <i>CV</i>	JMenning: <i>SM</i> <i>av1</i>	BMcCabe <i>SM</i> <i>BMcC</i>		RACapra <i>RGC</i>
DATE	5/13/93	5/13/93	5/13/93	5/17/93	5/24/93

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