

March 29, 2001

MEMORANDUM TO: James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Richard B. Ennis, Project Manager, Section 2 */RA/*
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: HOPE CREEK GENERATING STATION, FACSIMILE TRANSMISSION,
ISSUES TO BE DISCUSSED IN AN UPCOMING CONFERENCE CALL
(TAC NO. MB0644)

The attached information was transmitted by facsimile on March 29, 2001, to Mr. John Nagle of PSEG Nuclear LLC (the licensee). This information was transmitted to facilitate a upcoming conference call in order to determine an appropriate response time for the attached set of questions associated with the licensee's submittal dated December 1, 2000. In the submittal, the licensee requested a revision to the Hope Creek Generating Station (HCGS) Facility Operating License and Technical Specifications to increase the HCGS licensed power level by approximately 1.4%. This memorandum and the attachment do not convey or represent an NRC staff position regarding the HCGS power uprate request.

Docket No. 50-354

Attachment: Issues for Discussion in Upcoming Telephone Conference

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Issues for Discussion in Upcoming Telephone Conference
Related to PSEG License Change Request H00-05, dated December 1, 2000
Hope Creek Generating Station
1.4% Power Uprate

1. Attachment 1, Section 5.5, of the submittal provides the justification for the requested power uprate with respect to the design of the reactor coolant and balance-of-plant (BOP) piping. List the most critical BOP piping systems that were evaluated for the power uprate. Provide a summary of the evaluation used for BOP piping, components, and pipe supports, nozzles, penetrations, guides, valves, pumps, heat exchangers, and anchorage for pipe supports.

The regulatory basis for this question is that the BOP piping systems conform to the requirements of GDCs 1, 2, 4, 14, and 15 of Appendix A to 10 CFR Part 50 as they relate to maintaining structural integrity of pressure-retaining components and their supports (reference Standard Review Plan (SRP) Section 3.9.3).

2. Attachment 1, Section 5.11, of the submittal provides the justification for the requested power uprate with respect to the design of the control rod drive hydraulic system. Provide a summary of evaluation for the effects of the 1.4 percent power uprate on the design basis analysis of the control rod drive mechanism (CRDM). Confirm that the CRDMs structural integrity will be adequate for the 1.4 percent power uprate.

The regulatory basis for this question is that the CRDMs conform to the requirements of GDC 14 of Appendix A to 10 CFR Part 50 as it relates to maintaining the reactor coolant pressure boundary.

3. Discuss the functionality of safety-related mechanical components (i.e., all safety related valves and pumps, including air-operated valves (AOV) and power-operated relief valves) affected by the power uprate to demonstrate that the performance specifications and technical specification requirements (e.g., flow rate, close and open times) will be met for the proposed power uprate. Confirm that safety-related motor-operated valves (MOVs) in your Generic Letter (GL) 89-10 MOV program at HCGS will be capable of performing their intended function(s) following the power uprate including such affected parameters as fluid flow, temperature, pressure and differential pressure, and ambient temperature conditions. Please discuss effects of the proposed power uprate on the pressure locking and thermal binding of safety-related power-operated gate valves for GL 95-07 and on the evaluation of overpressurization of isolated piping segments for GL 96-06.

The regulatory basis for this question is that the assumptions, analyses, and conclusions of the HCGS programs associated with GL 89-10, GL 95-07, and GL 96-06 remain valid.