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March 21, 2001

U. S. Nuclear Regulatory Commission Washington D.C. 20555-0001 ATTENTION: Document Control Desk

- Subject: Duke Energy Corporation Catawba Nuclear Station, Units 1 and 2 Docket Nos. 50-413 and 414 Topical Report DPC-NE-3002, Revision 4
- Reference: 1) Letter, Duke Energy Corporation to U.S. Nuclear Regulatory Commission, ATTENTION: Document Control Desk, Dated April 19, 2000, SUBJECT: Topical Report DPC-NE-3002, Revision 4
 - 2) Letter, Duke Energy Corporation to U.S. Nuclear Regulatory Commission, ATTENTION: Document Control Desk, Dated August 24, 2000, SUBJECT: Topical Report DPC-NE-3002, Revision 4
 - 3) Letter, Duke Energy Corporation to U.S. Nuclear Regulatory Commission, ATTENTION: Document Control Desk, Dated September 22, 2000, SUBJECT: Topical Report DPC-NE-3002, Revision 4

In the letters referenced above, Duke Energy Corporation submitted proposed Revision 4 to Topical Report DPC-NE-3002, UFSAR Chapter 15 Transient Analysis Methodology. In order to support the NRC's review and approval of this topical report, Duke is submitting the attached Catawba Procedure EP/1/A/5000/E-3, Steam Generator Tube Rupture.

4001

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Please address any questions to J. S. Warren at (704) 382-4986.

Very truly yours,

M.S. Tucknam

M. S. Tuckman

Attachment

xc w/Attachment:

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Mr. D. J. Roberts NRC Senior Resident Inspector Catawba Nuclear Station U. S. Nuclear Regulatory Commission March 21, 2001 Page 3

bxc:

w/o Attachment

- G. B. Swindlehurst
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- T. Baumgardner
- M. H. Chernoff
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ELL

A. Purpose

This procedure provides actions to terminate leakage of reactor coolant into the secondary system following a steam generator tube rupture.

B. Symptoms or Entry Conditions

This procedure is entered from:

- a. EP/1/A/5000/E-0 (Reactor Trip Or Safety Injection), Step 25, when condenser air ejector radiation, S/G blowdown radiation or steamline radiation is abnormal.
- b. EP/1/A/5000/E-0 (Reactor Trip Or Safety Injection), Step 30, EP/1/A/5000/E-1 (Loss Of Reactor Or Secondary Coolant), Step 4, EP/1/A/5000/E-2 (Faulted Steam Generator Isolation), Step 10, EP/1/A/5000/ECA-2.1 (Uncontrolled Depressurization Of All Steam Generators), Step 7, and EP/1/A/5000/FR-H.3 (Response To Steam Generator High Level), Step 8, when secondary radiation is abnormal.
- c. EP/1/A/5000/E-0 (Reactor Trip Or Safety Injection), Step 29, EP/1/A/5000/E-1 (Loss Of Reactor Or Secondary Coolant), Step 3, EP/1/A/5000/ES-1.2 (Post LOCA Cooldown And Depressurization), Step 7, EP/1/A/5000/ES-3.1 (Post SGTR Cooldown Using Backfill), Step 5, EP/1/A/5000/ES-3.2 (Post SGTR Cooldown Using Blowdown), Step 5, EP/1/A/5000/ES-3.3 (Post SGTR Cooldown Using Steam Dump), Step 7, EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant Subcooled Recovery Desired), Step 11, and EP/1/A/5000/ECA-3.2 (SGTR With Loss Of Reactor Coolant Saturated Recovery Desired), Step 6, when a S/G N/R level increases in an uncontrolled manner.
- d. E-1 series foldout page whenever any S/G level increases in an uncontrolled manner or any S/G has abnormal radiation.

STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

C. Operator Actions

____ 1. Monitor Enclosure 1 (Foldout Page).

COOLDOWN.

STEAM GENERATOR TUBE RUPTURE

RESPONSE NOT OBTAINED ACTION/EXPECTED RESPONSE Perform the following: 2. Identify ruptured S/G(s) as follows: ____a. WHEN ruptured S/G(s) is identified, S/G level - INCREASING IN AN UNCONTROLLED MANNER. THEN perform Steps 3 through 6. OR b. GO TO Step 7. Chemistry or RP determines ruptured S/G by frisking the cation columns in the CT lab. OR The following EMF trip 1 lights - LIT: • 1EMF-26 (Steamline 1A) • 1EMF-27 (Steamline 1B) • 1EMF-28 (Steamline 1C) • 1EMF-29 (Steamline 1D). OR Chemistry determines ruptured S/G using 1EMF-34 (S/G Sample). OR • IF S/G Sampling is required to identify ruptured S/G(s), THEN: a. Ensure the following signals -RESET: ____1) Phase A Containment Isolations 2) CA System valve control 3) KC NC NI NM St signals. b. Align all S/Gs for Chemistry sampling. c. Notify Chemistry to sample all S/Gs for activity. Maintain one S/G available for NC Verify at least one intact S/G -3. **AVAILABLE FOR NC SYSTEM** System cooldown in subsequent steps.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4. Isolate steam flow from ruptured S/G(s) as follows:	
a. Verify all ruptured S/G(s) PORV - CLOSED.	 <u>WHEN</u> ruptured S/G(s) pressure is less than 1090 PSIG, <u>THEN</u> perform the following:
	1) Ensure ruptured S/G(s) PORV - CLOSED.
	2) <u>IF</u> ruptured S/G(s) PORV will not close, <u>THEN</u> manually close ruptured S/G(s) PORV isolation valve.
	3) IF ruptured S/G(s) PORV isolation valve will not manually close, THEN dispatch operator to close ruptured S/G(s) PORV isolation valve.
b. Verify S/G(s) 1B and 1C - INTACT.	b. Perform the following:
	1) IF CA Pump #1 is the only source of feedwater, <u>THEN</u> maintain steam flow to the CAPT from at least one S/G.
	2) IF S/G 1B is ruptured, THEN:
	a) Dispatch two operators to unlock and close 1SA-1 (Main Steam 1B To CAPT Maintenance Isol) (DH-624, FF-53, Rm 572) (Breakaway lock installed).
	 b) <u>IF</u> 1SA-1 cannot be closed, <u>THEN</u> dispatch two operators to unlock and close 1SA-3 (S/G 1B SM To CAPT Stop Check) (AB-551, DD-53, Rm 217) (Breakaway lock installed).
	(RNO continued on next page)

STEAM GENERATOR TUBE RUPTURE

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	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
4. (C	Continued)		
			3) IF S/G 1C is ruptured, THEN:
			a) Dispatch two operators to unlock and close 1SA-4 (Main Steam 1C To CAPT Maintenance Isol) (DH-624, FF-53, Rm 572) (Breakaway lock installed).
			 b) <u>IF</u> 1SA-4 cannot be closed, <u>THEN</u> dispatch two operators to unlock and close 1SA-6 (S/G 1C SM To CAPT Stop Check) (AB-551, DD-53, Rm 217) (Breakaway lock installed) (Ladder needed).
(Isolate blowdown and steam drain on a ruptured S/G(s) as follows: 	all	
	• S/G 1A:		
	1) Close 1SM-77A (S/G 1A Otlt Hd Bldwn C/V).	lr	 Dispatch operator to close 1SM-77A (S/G 1A Otlt Hdr Bldwn C/V) (DH-583, FF-GG, 43-44, Rm 591).
	 Verify the following blowdown isolation valves - CLOSED: 		
	a) 1BB-56A (S/G 1A Bldwn Cou Isol Insd).	nt	a) Manually close valve.
	b) 1BB-148B (S/G 1A Bldwn Cont Isol Byp).		b) Perform the following:
	Contrisor Bypy.		(1) Manually close valve.
			(2) IF valve will not close <u>AND</u> 1BB-56A is open, <u>THEN</u> dispatch operator to close 1BB-148B (S/G 1A Bldwn Cont Isol Byp) (DH-580, EE-FF, 44-45, Rm 591).

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4. (Continued)	
c) 1BB-57B (S/G 1A Bldwn Cont Isol Otsd).	c) Perform the following:(1) Manually close valve.
	(2) IF valve will not close <u>AND</u> 1BB-56A is open, <u>THEN</u> dispatch operator to close 1BB-57B (S/G 1A Bldwn Cont Isol Otsd) (DH-580, EE-FF, 44-45, Rm 591).
• S/G 1B:	
1) Close 1SM-76B (S/G 1B Otlt Hdr Bldwn C/V).	 Dispatch operator to close 1SM-76B (S/G 1B Otlt Hdr Bldwn C/V) (DH-583, FF-53, Rm 572).
 Verify the following blowdown isolation valves - CLOSED: 	
a) 1BB-19A (S/G 1B Bldwn Cont Isol Insd).	a) Manually close valve.
b) 1BB-150B (S/G 1B Bldwn Cont Isol Byp).	b) Perform the following:
	(1) Manually close valve.
	(2) <u>IF</u> valve will not close <u>AND</u> 1BB-19A is open, <u>THEN</u> dispatch operator to close 1BB-150B (S/G 1B Bldwn Cont Isol Byp) (DH-580, EE-FF, 52-53, Rm 572).
c) 1BB-21B (S/G 1B Bldwn Cont Isol Otsd).	c) Perform the following:
1301 01307.	(1) Manually close valve.
	(2) IF valve will not close AND 1BB-19A is open, THEN dispatch operator to close 1BB-21B (S/G 1B Bldwn Cont Isol Otsd) (DH-580, EE-FF, 52-53, Rm 572).

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4. (Continued)	
• S/G 1C:	
1) Close 1SM-75A (S/G 1C Otlt Hd Bldwn C/V).	Ir1) Dispatch operator to close 1SM-75A (S/G 1C Otlt Hdr Bldwn C/V) (DH-580, GG, 52-53, Rm 572).
 Verify the following blowdown isolation valves - CLOSED: 	
a) 1BB-60A (S/G 1C Bldwn Col Isol Insd).	nta) Manually close valve.
b) 1BB-149B (S/G 1C Bldwn Cont Isol Byp).	 b) Perform the following: (1) Manually close valve. (2) IF valve will not close AND 1BB-60A is open, THEN dispatch operator to close 1BB-149B (S/G 1C Bldwn Cont Isol Byp) (DH-578, FF-GG, 52-53, Rm 572).
c) 1BB-61B (S/G 1C Bldwn Co Isol Otsd).	nt c) Perform the following: (1) Manually close valve. (2) IF valve will not close <u>AND</u> 1BB-60A is open, <u>THEN</u> dispatch operator to close 1BB-61B (S/G 1C Bldwn Cont Isol Otsd) (DH-578, FF-GG, 52-53, Rm 572).

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ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
4. (Continued)		
• S/G 1D:		
1) Close 1SM-74B (S/G 1D Bldwn C/V).	Otlt Hdr	 Dispatch operator to close 1SM-74B (S/G 1D Otlt Hdr Bldwn C/V) (DH-583, FF-GG, 44-45, Rm 591).
 Verify the following blow isolation valves - CLOSE 		
a) 1BB-8A (S/G 1D Bld Isol Insd).	wn Cont	a) Manually close valve.
b) 1BB-147B (S/G 1D E Cont Isol Byp).	ldwn	b) Perform the following:(1) Manually close valve.
		(1) Indicative vill not close <u>AND</u> 1BB-8A is open, <u>THEN</u> dispatch operator to close 1BB-147B (S/G 1D Bldwn Cont Isol Byp) (DH-582, EE-FF, 43-44, Rm 591).
c) 1BB-10B (S/G 1D BI Isol Otsd).	dwn Cont	 c) Perform the following: (1) Manually close valve. (2) IF valve will not close AND 1BB-8A is open, THEN dispatch operator to close 1BB-10B (S/G 1D Bldwn Cont Isol Otsd) (DH-582, EE-FF, 43-44, Rm 591).

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
 Close the following valves on all uptite close clos	 Perform the following: a. Close the following valves on remaining S/Gs: - MSIV MSIV bypass valve. - MSIV bypass valve. - Diace steam dump control in manual and lower controller output to 0%. - C. Place "STEAM DUMP SELECT" switch in pressure mode. d. Transfer turbine steam seal supply to AS as follows: - 1) Open 1TL-8 (Aux Stm To Stm Seal Reg). - 2) Close 1TL-2 (Main Stm To Stm Seal Reg). 2) Close 1TL-2 (Main Stm To Stm Seal Reg). e. Ensure the following turbine S/V before seat drain valves - CLOSED: - 1 SM-41 (Stop VIv #1 Before Seat Drn) - 1 SM-43 (Stop VIv #2 Before Seat Drn) - 1 SM-42 (Stop VIv #3 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (Stop VIv #4 Before Seat Drn). - 1 SM-42 (MSRH 1A&1B SSRH Stm Source). - 1 HM-1 (MSRH 1A&1B SSRH Stm Source). - 1 HM-2 (MSRH 1C&1D SSRH Stm Source). (RNO continued on next page)

ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED 5. (Continued) - h. Dispatch operator to isolate steam flow from all ruptured S/G(s). - h. Dispatch operator to isolate steam flow from all ruptured S/G(s). - isolate steam flow From Ruptured S/G(s). - i. Use intact S/G(s) PORV for steam release. - j. IF at least one intact S/G cannot be isolated from all ruptured S/G(s). THEN GO TO E P1/IA/S000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired). 6. Control ruptured S/G(s) level as follows: - a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). a. Verify ruptured S/G(s) is also follows: - 1) IF any ruptured S/G is also faulted, THEN do not establish feed flow to the ruptured S/G(s) is not faulted or NC System cooldown. 2) IE any ruptured S/G(s) is not faulted or NC System cooldown, THEN: - a) Establish and maintain feed flow to affected S/G(s). - b) WHEN affected S/G(s). - b) WHEN affected S/G(s) N/R level and 6.c. - 3) GQ TO Step 7.
 h. Dispatch operator to isolate steam flow from all ruptured S/G(s). <u>REFER TO</u> Enclosure 2 (Locally Isolating Steam Flow From Ruptured S/G(s)). i. Use intact S/G(s) PORV for steam release. j. <u>IF</u> at least one intact S/G cannot be isolated from all ruptured S/G(s), <u>THEN</u> <u>GO TO EP/1/A/5000/ECA-3.1 (SGTR</u> With Loss Of Reactor Coolant - Subcooled Recovery Desired). 6. Control ruptured S/G(s) level as follows: a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). a. Perform the following:
 from all ruptured S/G(s). <u>REFER TO</u> Enclosure 2 (Locally Isolating Steam Flow From Ruptured S/G(s)). i. Use intact S/G cannot be isolated from all ruptured S/G(s). <u>THEN</u> <u>GO TO EP/1/A/5000/ECA-3.1 (SGTR</u> With Loss Of Reactor Coolant - Subcooled Recovery Desired). 6. Control ruptured S/G(s) level as follows: a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). i. J. J. F. any ruptured S/G is also faulted, <u>THEN</u> to not establish feed flow to the ruptured S/G(s) is not faulted <u>OR</u> is required for cooldown. 2) IF any ruptured S/G(s) is not faulted <u>OR</u> is required for cooldown. 2) IF any ruptured S/G(s) is not faulted <u>OR</u> is required for cooldown. THEN: a) Establish and maintain feed flow to affected S/G(s). b) <u>WHEN</u> affected S/G(s) N/R level greater than 16% (29% ACC), <u>THEN</u> perform Steps 6.b and 6.c.
 release. j. IE at least one intact S/G cannot be isolated from all ruptured S/G(s), THEN GO TO EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired). 6. Control ruptured S/G(s) level as follows: a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). 1) IF any ruptured S/G is also faulted, THEN do not establish feed flow to the ruptured S/G unless needed for NC System cooldown. 2) IF any ruptured S/G(s) is not faulted OR is required for cooldown, THEN: a) Establish and maintain feed flow to affected S/G(s). b) WHEN affected S/G(s) N/R level greater than 16% (29% ACC), THEN perform Steps 6.b and 6.c.
 isolated from all ruptured S/G(s), <u>THEN</u> <u>GO TO EP/1///S000/ECA-3.1 (SGTR</u> With Loss Of Reactor Coolant - Subcooled Recovery Desired). a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). a. Perform the following: 1) IF any ruptured S/G is also faulted, <u>THEN</u> do not establish feed flow to the ruptured S/G unless needed for NC System cooldown. 2) IF any ruptured S/G(s) is not faulted <u>OR</u> is required for cooldown, <u>THEN</u>: a) Establish and maintain feed flow to affected S/G(s). b) <u>WHEN</u> affected S/G(s) N/R level greater than 16% (29% ACC), <u>THEN</u> perform Steps 6.b and 6.c.
 a. Verify ruptured S/G(s) N/R level - GREATER THAN 16% (29% ACC). a. Perform the following: 1) IF any ruptured S/G is also faulted, <u>THEN</u> do not establish feed flow to the ruptured S/G unless needed for NC System cooldown. 2) IF any ruptured S/G(s) is not faulted <u>OR</u> is required for cooldown, <u>THEN:</u> a) Establish and maintain feed flow to affected S/G(s). b) <u>WHEN</u> affected S/G(s) N/R level greater than 16% (29% ACC), <u>THEN</u> perform Steps 6.b and 6.c.
 GREATER THAN 16% (29% ACC). 1) IF any ruptured S/G is also faulted, THEN do not establish feed flow to the ruptured S/G unless needed for NC System cooldown. 2) IF any ruptured S/G(s) is not faulted OR is required for cooldown, THEN: a) Establish and maintain feed flow to affected S/G(s). b) WHEN affected S/G(s) N/R level greater than 16% (29% ACC), THEN perform Steps 6.b and 6.c.

STEAM GENERATOR TUBE RUPTURE

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6. (Continued)	
 b. Isolate feed flow to all ruptured S/G(s) as follows: 	
• S/G 1A:	
1) Close 1CA-62A (CA Pmp A Disc	h 1) Perform the following:
To S/G 1A Isol).	a) Close 1CA-60 (CA Pump 1A Flow To S/G 1A).
	b) Dispatch operator to close 1CA-62A (CA Pmp A Disch To S/G 1A Isol) (DH-587, DD-EE, 44-45, Rm 591).
2) Close 1CA-66B (CA Pmp 1 Disc	h 2) Perform the following:
To S/G 1A Isol).	a) Close 1CA-64 (CA Pump #1 Flow To S/G 1A).
	b) Dispatch operator to close 1CA-66B (CA Pmp 1 Disch To S/G 1A Isol) (DH-584, DD-EE, 44-45, Rm 591).
• S/G 1B:	
1) Close 1CA-58A (CA Pmp A Disc	th 1) Perform the following:
To S/G 1B Isol).	a) Close 1CA-56 (CA Pump 1A Flow To S/G 1B).
	b) Dispatch operator to close 1CA-58A (CA Pmp A Disch To S/G 1B Isol) (DH-586, DD-EE, 52-53, Rm 572).
2) Close 1CA-54B (CA Pmp 1 Disc To S/G 1B Isol).	h 2) Perform the following:
10 3/G TB (Sol).	a) Close 1CA-52 (CA Pump #1 Flow To S/G 1B).
	b) Dispatch operator to close 1CA-54B (CA Pmp 1 Disch To S/G 1B Isol) (DH-584, DD-EE, 52-53, Rm 572).

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6. (Continued)	
• S/G 1C:	
1) Close 1CA-46B (CA Pmp B Disch	1) Perform the following:
To S/G 1C Isol).	a) Close 1CA-44 (CA Pump 1B Flow To S/G 1C).
	b) Dispatch operator to close 1CA-46B (CA Pmp B Disch To S/G 1C Isol) (DH-586, DD, 53-54, Rm 572).
2) Close 1CA-50A (CA Pmp 1 Disch	2) Perform the following:
To S/G 1C Isol).	a) Close 1CA-48 (CA Pump #1 Flow To S/G 1C).
	 b) Dispatch operator to close 1CA-50A (CA Pmp 1 Disch To S/G 1C Isol) (DH-584, EE-53, Rm 572).
• S/G 1D:	
1) Close 1CA-42B (CA Pmp B Disch	1) Perform the following:
To S/G 1D Isol).	a) Close 1CA-40 (CA Pump 1B Flow To S/G 1D).
	b) Dispatch operator to close 1CA-42B (CA Pmp B Disch To S/G 1D Isol) (DH-586, DD-EE, 43-44, Rm 591).
2) Close 1CA-38A (CA Pmp 1 Disch	2) Perform the following:
To S/G 1D Isol).	a) Close 1CA-36 (CA Pump #1 Flow To S/G 1D).
	 b) Dispatch operator to close 1CA-38A (CA Pmp 1 Disch To S/G 1D Isol) (DH-584, DD-EE, 43-44, Rm 591).

STEAM GENERATOR TUBE RUPTURE

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6. (Continued)	
c. IF <u>AT ANY TIME</u> ruptured S/G(s) N/R level is less than 16% (29% ACC), <u>THEN</u> perform Step 6.	
7. Verify Pzr PORV and isolation valve status as follows:	
a. Power to all Pzr PORV isolation valves - AVAILABLE.	a. Dispatch operator to restore power to affected Pzr PORV isolation valve(s):
	 1EMXD-F02C (Pressurizer Power Operated Relief Isol. Valve 1NC31B) (AB-560, BB-50, Rm 372)
	 1EMXC-F03C (Pressurizer Power Operated Relief Isol. Valve 1NC33A) (AB-577, BB-50, Rm 496)
	 1EMXD-F05A (Pressurizer Power Operated Relief Isol. Valve 1NC35B) (AB-560, BB-50, Rm 372).
b. All Pzr PORVs - CLOSED.	 b. <u>IF</u> Pzr pressure is less than 2315 PSIG, <u>THEN</u>:
	1) Manually close Pzr PORV(s).
	2) <u>IF</u> any Pzr PORV cannot be closed, <u>THEN</u> close its isolation valve.
	 <u>IF</u> Pzr PORV cannot be closed <u>OR</u> isolated, <u>THEN GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
c. At least one Pzr PORV isolation valve - OPEN.	 C. Open one Pzr PORV isolation valve unless it was closed to isolate an open Pzr PORV.
d. IF AT ANY TIME a Pzr PORV opens due to high pressure while in this procedure, THEN, after Pzr pressure decreases to less than 2315 PSIG, perform Step 7.b.	

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
 8. Verify main steamlines are intact as follows: All S/G pressures - STABLE OR INCREASING All S/Gs - PRESSURIZED. 	 IF pressure in any S/G is decreasing in an uncontrolled manner OR any S/G is depressurized, THEN: a. IF EP/1/A/5000/E-2 (Faulted Steam Generator Isolation) has been performed for all faulted S/G(s) not needed for NC System cooldown, THEN GO TO Step 9. b. IF faulted S/G(s) are not isolated AND faulted S/G(s) are not needed for NC System cooldown, THEN GO TO EP/1/A/5000/E-2 (Faulted Steam Generator Isolation).
 Control intact S/G levels as follows: a. Verify N/R level in all intact S/Gs - GREATER THAN 11% (29% ACC). 	 a. Perform the following: 1) Maintain total feed flow greater than 450 GPM to intact S/Gs until at least one intact S/G N/R level greater than 11% (29% ACC). 2) IF total feed flow greater than 450 GPM cannot be established, THEN contact station management for guidance to establish feed flow from one of the following alternate sources: CF CM Alternate low pressure water source.
 b. Throttle feed flow to maintain all intact S/G N/R levels between 11% (29% ACC) and 50%. c. Ensure CA suction source switchover criterion is monitored. <u>REFER TO</u> Enclosure 1 (Foldout Page). 	to increase in an uncontrolled manner, <u>THEN RETURN</u> <u>TO</u> Step 1.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
10. Ensure S/I - RESET: a. ECCS.	
b. D/G load sequencers.	 b. Dispatch operator to open the affected sequencer(s) control power breaker: 1EDE-F01F (Diesel Generator Load Sequencer Panel 1DGLSA) (AB-577, BB-46, Rm 496) 1EDF-F01F (Diesel Generator Load Sequencer Panel 1DGLSB) (AB-560, BB-46, Rm 372).
 c. <u>IF AT ANY TIME</u> a B/O occurs, <u>THEN</u> restart S/I equipment previously on. 11. Ensure the following containment 	bb-40, Rm 372).
isolation signals - RESET: • Phase A • Phase B.	
 12. Establish VI to containment as follows: - Ensure 1VI-77B (VI Cont Isol) - OPEN. - Verify VI pressure - GREATER THAN 85 PSIG. 	 Perform the following: a. Align N₂ to the Pzr PORVs by opening the following valves: - • 1NI-438A (Emer N2 From CLA A To 1NC-34A) - • 1NI-439B (Emer N2 From CLA B To 1NC-32B). b. IF VI pressure is less than 85 PSIG, THEN dispatch operator to ensure proper VI compressor operation.

[ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
13.	 Verify all AC busses are energized by offsite power as follows: A Train: "FTA B/O NORM FDR FRM ATC" - CLOSED "D/G 1A BKR TO ETA" - OPEN 1ETA - ENERGIZED. B Train: "FTB B/O NORM FDR FRM ATD" - CLOSED "D/G 1B BKR TO ETB" - OPEN 1ETB - ENERGIZED. 	 Perform the following: a. WHEN time allows, THEN attempt to restore offsite power while continuing with this procedure. <u>REFER TO</u> AP/1/A/5500/07 (Loss Of Normal Power). b. Manually start following equipment: Start all available CRD vent fans. Dispatch operator to start available VI compressors.
14.	Verify criteria to stop operating ND pumps as follows:	
_	_ a. At least one ND pump - ON.	a. <u>GO</u> <u>TO</u> Step 14.d.
_	b. NC pressure - GREATER THAN 285 PSIG.	b. <u>GO TO</u> Step 15.
	c. Ensure all ND pump(s) not supporting Cold Leg Recirc - STOPPED.	
_	d. IF AT ANY TIME NC pressure decreases to less than 285 PSIG in ar uncontrolled manner, <u>THEN</u> restart NE pumps.	
_ 15.	Verify ruptured S/G(s) - IDENTIFIED.	Do not continue in this procedure until ruptured S/G(s) identified.

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
16. 	Verify the following valves on all ruptured S/G(s) - CLOSED: - MSIV • MSIV bypass valves.	 Perform the following: a. Verify the following valves on at least one intact S/G - CLOSED: • MSIV • MSIV bypass valve. b. IF at least one intact S/G cannot be isolated from all ruptured S/G(s), <u>THEN</u> <u>GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
17.	Verify all ruptured S/G(s) pressure - GREATER THAN 320 PSIG.	<u>GO TO EP/1/A/5000/ECA-3.1 (SGTR</u> With Loss Of Reactor Coolant - Subcooled Recovery Desired).
18.	<u>WHEN</u> "P-11 PZR S/I BLOCK PERMISSIVE" status light (1SI-18) is lit <u>THEN</u> :	y
-	a. Depress ECCS steam pressure "BLOCK" pushbuttons.	
_	b. Verify main steam isolation blocked status lights (1SI-13) - LIT.	
	c. Maintain NC pressure less than 1955 PSIG using one of the following:	
	• Pzr spray OR	
	• Pzr PORV.	

STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- **NOTE** NC pump trip criteria based on NC subcooling does not apply after starting a controlled cooldown.
 - After the low steamline pressure main steam isolation signal is blocked, Main Steam Isolation will occur if the high steam pressure rate setpoint is exceeded.

19. Initiate NC System cooldown as follows:

____a. Determine required core exit temperature from the table below:

LOWEST RUPTURED S/G PRESSURE (PSIG)	CORE EXIT T/Cs (°F)
EQUAL TO OR GREATER THAN 1200	532 (512 ACC)
1100 - 1199	520 (501 ACC)
1000 - 1099	507 (489 ACC)
900 - 999	494 (476 ACC)
800 - 899	479 (461 ACC)
700 - 799	462 (445 ACC)
600 - 699	442 (426 ACC)
500 - 599	420 (405 ACC)
400 - 499	392 (379 ACC)
320 - 399	364 (352 ACC)

b. Verify the condenser is available as follows:

_____b. <u>GO TO</u> Step 19.e RNO.

- "C-9 COND AVAILABLE FOR STM DUMP" status light (1SI-18) - LIT
 - MSIV on intact S/G(s) OPEN.

STEAM GENERATOR TUBE RUPTURE

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ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

19. (Continued)

- ____ c. <u>WHEN</u> "P-12 LO-LO TAVG" status light (1SI-18) is lit, <u>THEN</u> place the steam dump interlock bypass switches in "BYP INTLK."
- ____ d. Verify steam dumps IN PRESSURE MODE.
- d. Place steam dumps in pressure mode as follows:
- ____1) Place "STM DUMP CTRL" in manual.
- ____ 2) Manually adjust the "STM DUMP CTRL" to match "STM DUMP CTRL" demand and "% STM DUMP DEMAND".
- ____3) <u>WHEN</u> demand on the "STM DUMP CTRL" is equal to the "% STM DUMP DEMAND", <u>THEN</u> place the steam dumps in pressure mode.

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
19. (Co	ontinued)		
· ·			
		 <u>GO TO EP/1/A/5000/ECA-3.1</u> (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired). <u>GO TO</u> Step 19.f. 	

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
1!	9. (Continued)	
_	f. Verify main steam isolation blocked status light (1SI-13) - LIT.	f. <u>IF</u> pressure in S/G(s) used for cooldown is approaching 800 PSIG, <u>THEN</u> :
		 Depressurize NC System to less than 1955 PSIG using one of the following:
		 Pzr spray
		OR
		• Pzr PORV.
		2) Maintain NC pressure less than 1955 PSIG.
_	g. Verify core exit T/Cs - LESS THAN REQUIRED TEMPERATURE.	g. <u>RETURN TO</u> Step 19.e.
_	h. Stabilize core exit T/Cs - LESS THAN REQUIRED TEMPERATURE.	
20.	Verify NC System cooldown in Step 19 COMPLETED.	 The NC System cooldown in Step 19 should be completed before continuing in this procedure.
21.	Verify ruptured S/G(s) pressure is unde operator control as follows:	r
_	_ a. All ruptured S/G(s) pressure - STABLE OR INCREASING.	a. Perform the following:
		 Ensure ruptured S/G(s) isolated. <u>REFER TO</u> Steps 3 through 6.
		 <u>IF</u> ruptured S/G(s) pressure is less than intact S/G(s) used for cooldown, <u>THEN GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
		(RNO continued on next page)

STEAM GENERATOR TUBE RUPTURE

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
21. (Continued)	
	 IF D/P between ruptured S/G(s) and intact S/G(s) used for cooldown is less than 250 PSIG, <u>THEN</u>:
	 Maintain total NC System cooldown less than 100°F in an hour.
	 Dump steam from intact S/Gs to maintain intact S/G pressures 250 PSIG below ruptured S/G(s) pressure.
	 4) IF intact S/G(s) used for cooldown can not be maintained at least 250 PSIG below the pressure of the ruptured S/G(s), <u>THEN GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
	5) IF AT ANY TIME while in this procedure the following cannot be maintained, <u>THEN RETURN</u> <u>TO</u> Step 21:
	 NC System cooldown less than 100°F in an hour.
	 Intact S/G pressures 250 PSIG below ruptured S/G(s) pressure.
	6) GO TO Step 22.
b. IF AT ANY TIME ruptured S/G(s) pressure is decreasing while in this procedure, THEN perform Step 21.	
22. Verify NC subcooling based on core ex T/Cs - GREATER THAN 20°F.	kit <u>IF NC subcooling cannot be promptly</u> restored to greater than 20°F, <u>THEN GO</u> <u>TO EP/1/A/5000/ECA-3.1 (SGTR With</u> Loss Of Reactor Coolant - Subcooled Recovery Desired).

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
23. Depressurize NC System using PZR Spray as follows:	
a. Verify normal Pzr spray flow - AVAILABLE.	a. <u>GO TO</u> Step 24.
b. Verify Pzr level - LESS THAN 76% (73% ACC)	b. Observe Caution prior to Step 26 and GO TO Step 26.
c. Depressurize NC System with maximur available spray.	n
d. Verify subcooling based on core exit T/Cs - GREATER THAN 0°F	d. <u>GO TO</u> Step 23.h.
e. Verify Pzr level - LESS THAN 76% (73% ACC)	e. <u>GO TO</u> Step 23.h.
f. Verify NC pressure - LESS THAN RUPTURED S/G(s) PRESSURE	 f. Perform the following: 1) IF spray valves are not effective in reducing NC pressure <u>OR</u> the ruptured S/G(s) N/R level is approaching 83%, <u>THEN GO TO</u> Step 24. 2) <u>RETURN TO</u> Step 23.d.
g. Verify Pzr level - GREATER THAN 11% (20% ACC).	 g. Perform the following: 1) <u>IF</u> spray valves are not effective in reducing NC pressure <u>OR</u> the ruptured S/G(s) N/R level is approaching 83%, <u>THEN GO TO</u> Step 24. 2) <u>RETURN TO</u> Step 23.d.

[ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
23	3. (Continued)	
	h. Close the following valve(s):	
	1) Pzr spray valves.	 1) Stop NC pump(s) supplying failed Pzr spray valve(s).
	2) 1NV-37A (NV Supply To Pzr Aux Spray).	 Ensure one of the following valves - CLOSED:
		 1NV-312A (Chrg Line Cont Isol)
		OR
		 1NV-314B (Chrg Line Cont Isol).
_	i. Observe Caution prior to Step 26 and <u>GO TO</u> Step 26.	
24.	Depressurize NC System using Pzr PORV as follows:	
_	a. Verify at least one Pzr PORV - AVAILABLE.	a. Establish NV aux spray as follows:
		1) Ensure at least one NI pump - ON.
		2) Ensure at least one NV pump - ON.
		 Ensure the following NV pump miniflow valves - OPEN:
		 1NV-203A (NV Pumps A&B Recirc Isol)
		 1NV-202B (NV Pmps A&B Recirc Isol).
		4) Close the following valves:
		 1NI-9A (NV Pmp C/L Inj Isol) 1NI-10B (NV Pmp C/L Inj Isol).
		5) Manually throttle 1NV-294 (NV Pmps A&B Disch Flow Ctrl) for 32 GPM charging line flow.
		6) Manually close 1NV-309 (Seal Water Injection Flow).
		(RNO continued on next page)

STEAM GENERATOR TUBE RUPTURE

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	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
24. (Co	ontinued)	
		7) Open the following valves:
		 1NV-312A (Chrg Line Cont Isol)
		 1NV-314B (Chrg Line Cont Isol).
		8) Place 1NV-309 in "AUTO".
		 Ensure the following valves - CLOSED:
		 1NC-27 (Pzr Spray Ctrl Frm Loop A)
		 1NC-29 (Pzr Spray Ctrl Frm Loop B)
		 1NV-39A (NV Supply To Loop D Isol)
		 1NV-32B (NV Supply To Loop A Isol).
		10) Maintain charging flow less than 180 GPM.
		11) Throttle 1NV-37A (NV Supply To Pzr Aux Spray) and charging flow as required.
		12) RETURN TO Step 23.b.
b.	Verify Pzr level - LESS THAN 76% (73% ACC)	b. Observe Caution prior to Step 26 and <u>GO TO</u> Step 26.
c.	Open one Pzr PORV.	
d.	Verify subcooling based on core exit T/Cs - GREATER THAN 0°F	d. <u>GO TO</u> Step 24.h.
e.	Verify Pzr level - LESS THAN 76% (73% ACC)	e. <u>GO</u> TO Step 24.h.
f.	Verify NC pressure - LESS THAN RUPTURED S/G(s) PRESSURE	f. RETURN TO Step 24.d.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
24. (Continued)	
g. Verify Pzr level - GREATER THAN 119 (20% ACC).	%g. <u>RETURN TO</u> Step 24.d.
h. Close Pzr PORV.	h. Close Pzr PORV isolation valve.
i. Close Pzr spray valve(s).	 i. Stop NC pump(s) supplying failed valve(s).
25. Verify NC pressure - INCREASING.	Perform the following:
	a. Close Pzr PORV isolation valve.
	b. <u>IF</u> pressure continues to decrease, <u>THEN GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
CAUTION S/I must be terminated when terminated overfilling the ruptured S/G(s). 26. Verify S/I termination criteria as follows	rmination criteria are satisfied to prevent
a. NC subcooling based on core exit T/Cs - GREATER THAN 0°F.	sa. <u>GO_TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
b. Verify secondary heat sink as follows:	b. <u>GO</u> TO EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant -
 N/R level in at least one intact S/G - GREATER THAN 11% (29% ACC) 	—
OR	
 Total feed flow available to S/G(s) - GREATER THAN 450 GPM. 	
c. NC pressure - STABLE OR INCREASING.	c. <u>GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
d. Pzr level - GREATER THAN 11% (20% ACC).	d. <u>RETURN TO</u> Step 17.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
27. Stop S/I pumps as follows:	
a. Stop NI pumps.	a. Perform the following:
	 IF NI Pump 1A failed to trip, THEN perform the following:
	a) Ensure the following valves - OPEN:
	 1NI-115A (NI Pump 1A Miniflow Isol) 1NI-147B (NI Pump Miniflow Hdr To FWST Isol).
	b) <u>WHEN</u> miniflow path aligned, <u>THEN</u> ensure the following valves - CLOSED:
	 1NI-121A (NI Pump 1A To H-Legs B&C) 1NI-118A (NI Pump 1A C-Leg Inj Isol).
	c) Dispatch operator to locally trip 1ETA#11 (1A NI Pump Motor) (AB-577, AA-49, Rm 496).
	(RNO continued on next page)

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ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
27. (Continued)	
	 <u>IF</u> NI Pump 1B failed to trip, <u>THEN</u> perform the following:
	a) Ensure the following valves - OPEN:
	 1NI-144A (NI Pump 1B Miniflow Isol) 1NI-147B (NI Pump Miniflow Hdr To FWST Isol).
	b) <u>WHEN</u> miniflow path aligned, <u>THEN</u> ensure the following valves - CLOSED:
	 1NI-150B (NI Pump 1B C-Leg Inj Isol) 1NI-152B (NI Pump 1B To H-Legs A&D).
	c) Dispatch operator to locally trip 1ETB#11 (1B NI Pump Motor) (AB-560, AA-49, Rm 372).
b. Ensure only one NV pump - ON.	
28. Verify VI pressure - GREATER THAN 50 PSIG.	In subsequent steps, control room control is lost for the following valves and local operation will be required:
	• 1NV-294 (NV Pmps A&B Disch Flow Ctrl
	 1NV-309 (Seal Water Injection Flow).
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STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE

29. Isolate NV S/I flowpath as follows:

- a. Verify the following valves OPEN:
- 1NV-252A (NV Pumps Suct From FWST)
- 1NV-253B (NV Pumps Suct From FWST).

RESPONSE NOT OBTAINED

- a. <u>IF</u> NV pump suctions are aligned for Cold Leg Recirc, <u>THEN</u>:
- ___1) Close 1NV-309 (Seal Water Injection Flow).
 - 2) <u>IF</u> control of 1NV-309 is lost from the control room, <u>THEN</u>:
 - ____a) Place the controller for 1NV-309 in the 100% demand position.
 - ____b) Dispatch operator with a radio to close 1NV-308 (Seal Wtr Inj Flow Ctrl Isol) (AB-554, JJ-54, Rm 233) (Ladder needed).
 - ____ c) Throttle 1NV-308 to control seal injection flow as required in subsequent steps.
 - 3) Open the following valves:
 - 1NV-312A (Chrg Line Cont Isol)
 - 1NV-314B (Chrg Line Cont Isol).
 - IF 1NV-312A AND 1NV-314B cannot be opened, THEN dispatch operator to open the affected valve(s):
 - 1NV-312A (Chrg Line Cont Isol) (AB-553, HH-JJ, 52, Rm 217) (Ladder needed)
 - 1NV-314B (Chrg Line Cont Isol) (AB-553, GG-HH, 52-53, Rm 217) (Ladder needed).
- _____5) Do not continue in this procedure until 1NV-312A and 1NV-314B are open.
- ____ 6) IF NC pressure is greater than 1950 PSIG, <u>THEN</u> throttle 1NV-309 or 1NV-308 to 50% open.

(RNO continued on next page)

STEAM GENERATOR TUBE RUPTURE

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	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED		
29. (Continued)					
			7) Open 1NV-294 (NV Pmps A&B Disch Flow Ctrl).		
			 IF control of 1NV-294 is lost from the control room, <u>THEN</u>: 		
			a) Place the controller for 1NV-294 in the 100% demand position.		
			 b) Dispatch operator with a radio to throttle 1NV-295 (NV Pmps A & B Disch Ctrl Isol) (AB-551, JJ-55, Rm 231) to control charging flow as required in subsequent steps. 		
			9) Close the following valves:		
			 1NI-9A (NV Pmp C/L Inj Isol) 1NI-10B (NV Pmp C/L Inj Isol). 		
			 IF 1NI-9A AND 1NI-10B cannot be closed, THEN dispatch operator to close affected valve(s): 		
			 1NI-9A (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A) 		
			 - 1NI-10B (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A). 		
			 Throttle charging and seal injection to maintain the following: 		
			 Charging line flow between 60 GPM and 180 GPM 		
			 NC pump seal injection flow. 		
		-	12) GO TO Step 31.		

STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE

29. (Continued)

- b. Verify the following valves OPEN:
 - 1NV-203A (NV Pumps A&B Recirc Isol)
 - 1NV-202B (NV Pmps A&B Recirc Isol).

RESPONSE NOT OBTAINED

- b. Perform the following:
- ____1) Open affected valve(s).
- ____ 2) IF 1NV-203A AND 1NV-202B are open, THEN GO TO Step 29.c.
 - Dispatch operator to open affected valve(s):
 - 1NV-203A (NV Pumps A&B Recirc Isol) (AB-554, HH-JJ, 54-55, Rm 231) (Ladder needed)
 - 1NV-202B (NV Pmps A&B Recirc Isol) (AB-554, HH-JJ, 54-55, Rm 231) (Ladder needed).
- ____ 4) Close 1NV-309 (Seal Water Injection Flow).
 - 5) IF control of 1NV-309 is lost from the control room, THEN:
 - _____a) Place the controller for 1NV-309 in the 100% demand position.
 - b) Dispatch operator with a radio to close 1NV-308 (Seal Wtr Inj Flow Ctrl Isol) (AB-554, JJ-54, Rm 233) (Ladder needed).
 - ___ c) Throttle 1NV-308 to control seal injection flow as required in subsequent steps.
 - 6) Open the following valves:
 - 1NV-312A (Chrg Line Cont Isol)
 - 1NV-314B (Chrg Line Cont Isol).

(RNO continued on next page)

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
29. (Continued)	
	 IF 1NV-312A AND 1NV-314B cannot be opened, THEN dispatch operator to open the affected valve(s):
	 1NV-312A (Chrg Line Cont Isol) (AB-553, HH-JJ, 52, Rm 217) (Ladder needed)
	 1NV-314B (Chrg Line Cont Isol) (AB-553, GG-HH, 52-53, Rm 217) (Ladder needed).
	8) Do not continue in this procedure until 1NV-312A and 1NV-314B are open.
	9) IF NC pressure is greater than 1950 PSIG, <u>THEN</u> throttle 1NV-309 or 1NV-308 to 50% open.
	10) Open 1NV-294 (NV Pmps A&B Disch Flow Ctrl).
	 IF control of 1NV-294 is lost from the control room, <u>THEN</u>:
	a) Place the controller for 1NV-294 in the 100% demand position.
	 b) Dispatch operator with a radio to throttle 1NV-295 (NV Pmps A & B Disch Ctrl Isol) (AB-551, JJ-55, Rm 231) to control charging flow as required in subsequent steps.
	12) Close the following valves:
	• 1NI-9A (NV Pmp C/L Inj Isol)
	● 1NI-10B (NV Pmp C/L Inj Isol).
	(RNO continued on next page)

STEAM GENERATOR TUBE RUPTURE

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	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED			
29. (Continued)					
		 <u>IF</u> 1NI-9A <u>AND</u> 1NI-10B cannot be closed, <u>THEN</u> dispatch operator to close affected valve(s): 			
		 1NI-9A (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A) 			
		 1NI-10B (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A). 			
		14) Throttle charging and seal injection to maintain the following:			
		 Charging line flow between 60 GPM and 180 GPM 			
		 NC pump seal injection flow. 			
		15) <u>WHEN</u> 1NV-203A <u>AND</u> 1NV-202B are opened, <u>THEN</u> charging flow may be reduced below 60 GPM.			
		16) GO TO Step 31.			
	c. Close the following valves:	 Dispatch operator to close affected valve(s): 			
	 1NI-9A (NV Pmp C/L Inj Isol) 1NI-10B (NV Pmp C/L Inj Isol). 	 1NI-9A (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A) 			
		 1NI-10B (NV Pmp C/L Inj Isol) (AB-570, JJ, 51-52, Rm 318A). 			
30.	Establish charging as follows:				
	a. Throttle 1NV-294 (NV Pmps A&B Disc Flow Ctrl) for 32 GPM charging line flow.	h a. Perform the following:			
		1) Place the controller for 1NV-294 in the 100% demand position.			
		 2) Dispatch operator with a radio to throttle 1NV-295 (NV Pmps A & B Disch Ctrl Isol) (AB-551, JJ-55, Rm 231) for 32 GPM charging line flow. 			
		(RNO continued on next page)			

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	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
30. (Co	ontinued)	
		 3) Throttle 1NV-295 to control charging flow as required in subsequent steps.
b.	_ b. Close 1NV-309 (Seal Water Injection Flow).	b. Perform the following:
		 1) Place the controller for 1NV-309 in the 100% demand position.
		 2) Dispatch operator with a radio to throttle 1NV-308 (Seal Wtr Inj Flow Ctrl Isol) (AB-554, JJ-54, Rm 233) (Ladder needed) to maintain 32 GPM total seal water flow in subsequent steps.
C.	Ensure one of the following valves - OPEN:	
	• 1NV-32B (NV Supply To Loop A Isc	DI)
	OR	
	• 1NV-39A (NV Supply To Loop D Isol).	
d.	Open the following valves:	 Dispatch operator to open the affected valve(s):
_	 1NV-312A (Chrg Line Cont Isol) 1NV-314B (Chrg Line Cont Isol). 	 1NV-312A (Chrg Line Cont Isol) (AB-553, HH-JJ, 52, Rm 217) (Ladder needed)
		 1NV-314B (Chrg Line Cont Isol) (AB-553, GG-HH, 52-53, Rm 217) (Ladder needed).
e.	Verify 1NV-309 - ABLE TO BE OPERATED FROM THE CONTROL ROOM.	e. <u>GO TO</u> Step 30.g.
f.	Place 1NV-309 in "AUTO".	
g.	Maintain charging flow less than 180 GPM.	

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
31. Control charging flow to maintain Pzr level greater than 11% (20% ACC).	
32. Verify S/I flow not required as follows:	
a. NC subcooling based on core exit T/Cs - GREATER THAN 0°F.	s a. Perform the following:
	 1) Manually start S/I pumps and align valves as necessary to restore NC subcooling.
	2) <u>GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).
b. Pzr level - GREATER THAN 11% (20% ACC).	b. Perform the following:
	 1) Control charging flow to restore Pzr level to greater than 11% (20% ACC).
	 <u>IF</u> Pzr level cannot be maintained at greater than 11% (20% ACC), <u>THEN</u>:
	a) Manually start S/I pumps and align valves as necessary to restore Pzr level.
	b) <u>GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant - Subcooled Recovery Desired).

STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE

33.	Verify proper NS pump operation as
	follows:

- a. At least one NS pump ON.
- b. Verify the following valves OPEN:
 - 1FW-27A (ND Pump 1A Suct From FWST)
- 1FW-55B (ND Pump 1B Suct From FWST).
- _ c. Containment pressure LESS THAN 2.4 PSIG.

RESPONSE NOT OBTAINED

- ____ a. GO TO Step 34.
 - b. Perform the following:
 - ____1) <u>WHEN</u> containment pressure is less than 1 PSIG, <u>THEN</u> perform Steps 33.d through 33.f.
 - ____ 2) GO TO Step 34.
 - c. Perform the following:
 - <u>WHEN</u> containment pressure is less than 2.4 PSIG, <u>THEN</u> perform Step 33.
 - ____ 2) GO TO Step 34.

- _ d. Reset NS.
- ____e. Stop NS pumps.
 - f. Close the following valves:
 - INS-29A (NS Spray Hdr 1A Cont Isol)
 - 1NS-32A (NS Spray Hdr 1A Cont Isol)
 - 1NS-15B (NS Spray Hdr 1B Cont Isol)
 - 1NS-12B (NS Spray Hdr 1B Cont Isol).

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[ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
34.	Ensure proper operation of VCT Makeup Control System as follows:	
_	a. Determine the required shutdown boron concentration. <u>REFER TO</u> ROD Book, Section 5.11.	
	b. <u>WHEN</u> the required shutdown boron concentration is determined, <u>THEN</u> :	
	 Adjust VCT makeup controls for a boron concentration that is greater than or equal to the required shutdown boron concentration. 	
	2) Ensure "NC MAKEUP MODE SELECT" - IN "AUTO".	
	3) Place the "NC MAKEUP CONTROL" switch momentarily to the "START" position.	
35.	Establish normal letdown as follows:	
	a. Verify VI pressure - GREATER THAN 35 PSIG.	a. Perform the following:
		 <u>WHEN</u> VI pressure is greater than 35 PSIG, <u>THEN</u> perform Steps 35.b through 35.n.
		2) GO TO Step 36.
_	_ b. Verify Pzr level - GREATER THAN 25% (34% ACC).	b. Perform the following:
		 <u>WHEN</u> Pzr level increases to greater than 25% (34% ACC), <u>THEN</u> perform Steps 35.c through 35.n.
		2) GO TO Step 36.
	c. Ensure the following valves - CLOSED:	
	• 1KC-56A (KC To ND Hx 1A Sup Isol)	
	• 1KC-81B (KC To ND Hx 1B Sup Isol).	

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
35. (Continued)	
 d. Verify the following EMF trip 1 lights - DARK: 	d. Perform the following:
 1EMF-53A (Containment Trn A) 1EMF-53B (Containment Trn B). 	 1) Notify station management to evaluate restoring normal letdown with high NC System activity.
	2) Establish excess letdown. <u>REFER</u> <u>TO</u> Enclosure 4 (Establishing Excess Letdown).
	3) <u>WHEN</u> station management approval to establish normal letdown is obtained, <u>THEN</u> perform Steps 35.e through 35.n.
	4) GO TO Step 36.
 e. Verify the following values for the operating KC train(s) - OPEN: 	e. Manually open valve(s).
Train A:	
 1KC-1A (Aux Bldg Non-Ess Ret Hdr Isol) 	
 1KC-50A (Aux Bldg Non-Ess Hdr Isol). 	
Train B:	
 1KC-2B (Aux Bldg Non-Ess Ret Hdr Isol) 	
 1KC-53B (Aux Bldg Non-Ess Hdr Isol). 	
f. While performing the following steps, adjust 1NV-148 (Letdn Press Control) to maintain letdown pressure at 350 PSIG.	
g. Ensure 1NV-849 (Letdn Flow Var Orif Ctrl) valve demand position - 0%.	

STEAM GENERATOR TUBE RUPTURE

RESPONSE NOT OBTAINED ACTION/EXPECTED RESPONSE 35. (Continued) h. Perform the following: h. Open the following valves: 1) Ensure the following valves -• 1NV-1A (NC Letdn To Regen Hx Isol) CLOSED: 1NV-2A (NC Letdn To Regen Hx Isol) • 1NV-1A (NC Letdn To Regen Hx • 1NV-15B (Letdn Cont Isol). Isol) • 1NV-2A (NC Letdn To Regen Hx Isol) • 1NV-15B (Letdn Cont Isol). 2) Establish excess letdown. REFER TO Enclosure 4 (Establishing Excess Letdown). 3) **GO TO** Step 36. While performing the following steps, _ i. manually adjust charging flow to maintain letdown subcooled. ____j. Open 1NV-10A (Letdn Orif 1B Otlt Cont Isol). k. Throttle 1NV-849 (Letdn Flow Var Orif Ctrl) until valve demand position is 10% open. I. Monitor letdown flow and letdown pressure.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
35. (Continued)	
m. Verify letdown flow and letdown pressure - INCREASES.	m. Perform the following:
pressure - INONEASES.	 IF 1NV-849 valve demand position is 100% open, THEN:
	a) Close the following valves:
	 1NV-849 (Letdn Flow Var Orif Ctrl)
	 1NV-10A (Letdn Orif 1B Otlt Cont Isol)
	 1NV-1A (NC Letdn To Regen Hx Isol)
	 1NV-2A (NC Letdn To Regen Hx Isol).
	b) Establish excess letdown. <u>REFER</u> <u>TO</u> Enclosure 4 (Establishing Excess Letdown).
	c) <u>GO</u> <u>TO</u> Step 36.
	 Throttle open 1NV-849 until one of the following conditions are met:
	 Letdown flow and letdown pressure increases
	OR
	 Valve demand position increases by 10%
	OR
	 Valve demand position is 100% open.
	3) RETURN TO Step 35.I.

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STEAM GENERATOR TUBE RUPTURE

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
35. (Continued)	
n. WHEN 5 minutes has passed, THEN:	
 Adjust 1NV-849 (Letdn Flow Var Orif Ctrl) to attempt to increase letdown flow to greater than 30 GPM. 	
2) Open one of the following valves:	2) Perform the following:
INV-13A (Letdn Orif 1A Otlt Cont Isol) Isol Isol	a) Close the following valves:
OR	 1NV-849 (Letdn Flow Var Orif Ctrl)
 1NV-11A (Letdn Orif 1C Otlt Cont Isol). 	 1NV-10A (Letdn Orif 1B Otlt Cont Isol)
	 1NV-1A (NC Letdn To Regen Hx Isol)
	 1NV-2A (NC Letdn To Regen Hx Isol).
	b) Establish excess letdown. <u>REFER</u> <u>TO</u> Enclosure 4 (Establishing Excess Letdown).
	c) <u>GO TO</u> Step 36.
3) Close 1NV-10A (Letdn Orif 1B Otlt Cont Isol).	
4) Close 1NV-849 (Letdn Flow Var Orif Ctrl).	
5) Ensure 1NV-148 (Letdn Press Control) - IN "AUTO".	

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
36.	Align NV pump suction to the VCT as follows:	
	 a. Verify at least one of the following NV pump suction valves - OPEN: 1NV-252A (NV Pumps Suct From FWST) OR 1NV-253B (NV Pumps Suct From FWST). b. Open the following valves: 1NV-188A (VCT Oth Isol) 1NV-189B (VCT Oth Isol). c. Close the following valves: 1NV-252A (NV Pumps Suct From FWST) 1NV-253B (NV Pumps Suct From FWST) 	 a. Perform the following: 1) Notify station management for guidance to restore NV pump suction to the VCT. 2) <u>GO TO</u> Step 37.
 FWST). NOTE Enclosure 5 (NC Pressure And Makeup Control to Minimize Leakage) shall remain in effect until subsequent procedures provide alternative NC pressure and makeup control guidance. 37. Control NC pressure and charging flow to minimize primary to secondary leakage. <u>REFER TO</u> Enclosure 5 (NC Pressure And Makeup Control to Minimize Leakage). 		

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
38.	Verify conditions to stop operating D/Ge as follows:	5
	_ a. At least one D/G - ON.	a. <u>GO TO</u> Step 39.
	 b. Verify 1ETA is energized by offsite power as follows: 	b. Perform the following:1) Attempt to restore offsite power to
	• "D/G 1A BKR TO ETA" - OPEN	affected switchgear. <u>REFER TO</u> AP/1/A/5500/07 (Loss Of Normal
	• 1ETA - ENERGIZED.	Power).
		2) <u>GO TO</u> Step 38.d.
_	 c. Dispatch operator to stop 1A D/G and place in standby readiness. <u>REFER</u> <u>TO</u> OP/1/A/6350/002 (Diesel Generato Operation). 	ır
	 Verify 1ETB is energized by offsite power as follows: 	d. Perform the following:
	• "D/G 1B BKR TO ETB" - OPEN	1) Attempt to restore offsite power to affected switchgear. <u>REFER TO</u> AP/1/A/5500/07 (Loss Of Normal
	• 1ETB - ENERGIZED.	Power).
		2) GO TO Step 39.
-	e. Dispatch operator to stop 1B D/G and place in standby readiness. <u>REFER</u> <u>TO</u> OP/1/A/6350/002 (Diesel Generato Operation).)r

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[ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
39.	Minimize secondary system contamination as follows:	
	a. Dispatch operators to perform the following:	
	 1) Inspect aux building and turbine building for leakage. 	
	2) Isolate or minimize leakage where possible but do not isolate S/I or charging paths to the NC System.	
	— 3) Isolate or identify leakage into the turbine building sump. <u>REFER TO</u> PT/1/B/4150/001G (Turbine Building Sump Isolation).]
_	_ b. Ensure the CM polishing demineralizers - IN SERVICE.	;
-	 c. Align auxiliary systems to minimize secondary side contamination. <u>REFER</u> <u>TO</u> Enclosure 6 (Auxiliary System Alignment). 	•
40.	Operate Pzr heaters as necessary to saturate Pzr water at ruptured S/G(s) pressure.	

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
41.	Verify NC pump cooling is aligned as follows:	
	a. Verify all of the following KC valves - OPEN:	 Perform one of the following based on seal injection status:
	• 1KC-230A (Rx Bldg Non-Ess Hdr Isol)	• IF NC pump seal injection flow is greater than 6 GPM to each NC
	 1KC-3A (Rx Bldg Non-Ess Ret Hdr Isol) 	pump, THEN manually open the affected valve(s).
	 1KC-394A (NC Pump 1A Therm Bar Otlt) 	 IF NC pump seal injection flow is less than 6 GPM to any NC pump, THEN:
	 1KC-425A (NC Pumps Ret Hdr Cont Isol) 	1) Ensure 1KC-425A (NC Pumps Ret Hdr Cont Isol) - CLOSED.
	 1KC-345A (NC Pump 1C Therm Bar Otlt) 	2) Manually open the other affected valve(s).
	 1KC-228B (Rx Bldg Non-Ess Hdr Isol) 	3) <u>WHEN</u> all other values are open, <u>THEN</u> dispatch operator to slowly establish KC flow to NC pump
	 1KC-18B (Rx Bldg Non-Ess Ret Hdr Isol) 	thermal barriers by opening 1KC-425A (NC Pumps Ret Hdr Cont Isol) (AB-588, GG-52, Rm
	 1KC-364B (NC Pump 1B Therm Bar Otit) 	419) (Ladder needed).
	— • 1KC-338B (NC Pumps Sup Hdr Cont Isol)	
	• 1KC-424B (NC Pumps Ret Hdr Cont Isol)	
	 1KC-413B (NC Pump 1D Therm Bar Otlt). 	
_	_ b. NC pump seal injection flow - GREATER THAN 6 GPM TO EACH NC PUMP.	b. <u>WHEN</u> seal leakoff temperatures are less than 235°F, <u>THEN</u> slowly throttle 1NV-309 (Seal Water Injection Flow) or 1NV-308 (Seal Wtr Inj Flow Ctrl Isol) to establish 32 GPM total seal water flow.

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
42. Establish NC pump seal return flow as follows:	
a. Verify NC pump seal injection flow - GREATER THAN 6 GPM TO EACH NC PUMP.	 a. Perform the following: 1) <u>WHEN</u> NC pump seal injection is restored, <u>THEN</u> perform Steps 42.b through 42.g. 2) <u>GO TO</u> Step 43.
b. Verify 1AD-7, D/1 "SEALWATER HX KC HI/LO FLOW" - DARK.	 b. Perform the following: 1) Notify station management to evaluate restoring NC pump seal return flow. 2) <u>WHEN</u> notified by station management <u>OR</u> 1AD-7, D/1 dark, <u>THEN</u> perform Steps 42.c through 42.g. 3) <u>GO TO</u> Step 43.
 c. Verify the following EMF trip 1 lights - DARK: 1EMF-53A (Containment Trn A) 1EMF-53B (Containment Trn B). 	 c. Perform the following: 1) Notify station management to evaluate restoring NC pump seal return with high NC System activity.
	 <u>WHEN</u> station management approval to establish NC pump seal return is obtained, <u>THEN</u> perform Steps 42.d through 42.g. <u>GO</u> <u>TO</u> Step 43.

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	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4	2. (Continued)	
-	d. Verify NCDT pressure - LESS THAN VCT PRESSURE.	d. Perform the following:
	VOT THEOREM.	 1) Consult with station management to establish normal NCDT pressure. <u>REFER TO</u> OP/1/A/6500/014 (Operations Controlled Liquid Waste Systems)
		 2) <u>WHEN</u> NCDT pressure is less than VCT pressure, <u>THEN</u> perform Steps 42.e through 42.g.
		3) GO TO Step 43.
	e. Open the following valves:	
	 1NV-89A (NC Pmps Seal Ret Cont Isol) 	
	 1NV-91B (NC Pmps Seal Ret Cont Isol). 	
	f. IF AT ANY TIME NCDT pressure is greater than VCT, <u>THEN</u> perform the following:	
	1) Monitor NC Pump #1 seal ΔP .	
	2) Verify excess letdown - ISOLATED	D 2) Align 1NV-125B (Excess Letdn Hx Otlt Ctrl) to "NCDT".
	3) Close the following valves:	
	 1NV-89A (NC Pmps Seal Ret Cont Isol) 	
	 1NV-91B (NC Pmps Seal Ret Cont Isol). 	
-	g. Verify excess letdown - ISOLATED.	g. Align 1NV-125B (Excess Letdn Hx Otlt Ctrl) to "VCT".

.

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
NOTE	Preference should be given to runnir provide Pzr spray capability.	ng NC Pump 1B and then NC Pump 1A to
43. Ve	rify NC pump status as follows:	
a.	At least one NC pump - ON.	a. Perform the following:
		1) Attempt to start one NC pump. <u>REFER TO</u> Enclosure 7 (NC Pump Start).
		2) Verify Natural Circulation until an NC pump can be started. <u>REFER</u> <u>TO</u> Enclosure 8 (Natural Circulation Monitoring Parameters).
		3) GO TO Step 44.
b.	Ensure only one NC pump - ON.	
c.	Ensure the normal Pzr spray valve associated with secured NC pump is manual and closed.	in
44. De	termine status of N/Is as follows:	
a.	Verify I/R channels - LESS THAN 10 ⁻¹⁰ AMPS.	a. Perform the following:
	10 1º AMPS.	1) <u>WHEN</u> I/R channels are less than 10 ⁻¹⁰ Amps, <u>THEN</u> perform Steps 44.b and 44.c.
		2) GO TO Step 45.
b.	Verify S/R channels - ENERGIZED.	b. Place S/R select switches in "RESET".
C.	Transfer one channel of the "NIS RECORDER" to S/R instrumentation.	l.

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
45.	Shutdown unnecessary plant equipmen as follows:	nent	
	a. Ensure the following breakers and MODs - OPEN:		
	• "GEN DE-EXC BKR" (relay)	 Notify Power Delivery. 	
	 "EXC FIELD BKR" MOD 1BG and 1BT MOD 1AG and 1AT Generator Breakers 1A and 1B. 		
_	_ b. Verify "MAN/AUTO REG" select switcl "MAN" mode light - LIT.	itch b. Transfer to manual mode.	
_	c. Dispatch operator to secure NF chiller and pumps.	lers	
	_ d. Stop excess condensate booster pumps.		
_	_ e. Stop excess hotwell pumps.		
_	_ f. Stop C heater drain pumps.		
_	g. Stop excess RC pumps and cooling tower fans. <u>REFER TO</u> OP/1/B/6400/001A (Condenser Circulating Water).	J	

STEAM GENERATOR TUBE RUPTURE

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	ACTION/EXPECTED RESPONSE]	RESPONSE	NOT OBTAINED)	
46.	Consult with station management to determine appropriate post-SGTR cooldown method:					
	• <u>GO TO</u> EP/1/A/5000/ES-3.1 (Post - SGTR Cooldown Using Backfill)					
	OR					
	• <u>GO TO</u> EP/1/A/5000/ES-3.2 (Post - SGTR Cooldown Using Blowdown)					
	OR					
_	 <u>GO TO</u> EP/1/A/5000/ES-3.3 (Post - SGTR Cooldown Using Steam Dump). 					
		<u>END</u>				

Enclosure 1 - Page 1 of 2 Foldout Page

1. NC Pump Trip Criteria:

- IF the following conditions are satisfied, THEN trip all NC pumps while maintaining seal injection flow:
 - At least one NV or NI pump ON
 - NC subcooling based on core exit T/Cs LESS THAN OR EQUAL TO 0°F.
- 2. Position Criteria for 1NV-202B and 1NV-203A (NV Pumps A&B Recirc Isol):
 - IF NC pressure is less than 1500 PSIG AND NV S/I flowpath is aligned, THEN close 1NV-202B and 1NV-203A.
 - IF NC pressure is greater than 2000 PSIG, THEN open 1NV-202B and 1NV-203A.
- 3. S/I Reinitiation Criteria:
- **NOTE** The following criteria is not applicable until after Step 27 is completed.
 - <u>IF</u> NC subcooling based on core exit T/Cs is less than 0°F <u>OR</u> Pzr level cannot be maintained greater than 11% (20% ACC), <u>THEN</u>:
 - a. Manually start S/I pumps and align valves as necessary to restore subcooling and Pzr level.
 - b. <u>GO TO</u> EP/1/A/5000/ECA-3.1 (SGTR With Loss Of Reactor Coolant Subcooled Recovery Desired).

4. Secondary Integrity Criteria:

<u>IF</u> pressure in any unisolated S/G is decreasing in an uncontrolled manner <u>OR</u> any unisolated S/G has completely depressurized, <u>THEN GO TO EP/1/A/5000/E-2</u> (Faulted Steam Generator Isolation) unless needed for NC System cooldown.

5. Cold Leg Recirc Switchover Criterion:

• IF FWST level decreases to 37% (1AD-9, D/8 "FWST 2/4 LO LEVEL" lit), THEN GO TO EP/1/A/5000/ES-1.3 (Transfer To Cold Leg Recirculation).

Enclosure 1 - Page 2 of 2 Foldout Page

6. CA Suction Source Switchover Criteria:

- IF either of the following annunciators are lit, THEN REFER TO AP/1/A/5500/06 (Loss Of S/G Feedwater).
 - 1AD-5, H/4 "CACST LO LEVEL"

OR

• 1AD-8, B/1 "UST LO LEVEL"

7. Multiple Tube Rupture Criteria:

- IF level in any intact S/G increases in an uncontrolled manner OR any intact S/G indicates abnormal radiation, THEN:
 - a. Stop any operator controlled cooldown and depressurization in progress.
 - b. RETURN TO EP/1/A/5000/E-3 (Steam Generator Tube Rupture), Step 1.

Enclosure 2 - Page 1 of 1 Locally Isolating Steam Flow From Ruptured S/G(s)

1. Close 1SA-22 (Main Stm To CSAE) (TB-594, 1M-32).

2. Close the following valves:

- 1SM-166 (Main Turb S/V #1 Continuous Drn Orif 0-34 Inlet) (TB-594, 1H-32)
- 1SM-168 (Main Turb S/V #2 Continuous Drn Orif 0-35 Inlet) (TB-594, 1H-32)
- 1SM-170 (Main Turb S/V #3 Continuous Drn Orif 0-36 Inlet) (TB-594, 1H-32)
- 1SM-172 (Main Turb S/V #4 Continuous Drn Orif 0-37 Inlet) (TB-594, 1H-32)
- 1SM-154 (Main Turb S/V #1 Auto Drn Vlv Inlet Isol) (TB-594, 1H-32)
- 1SM-157 (Main Turb S/V #2 Auto Drn VIv Inlet Isol) (TB-594, 1H-32)
- 1SM-160 (Main Turb S/V #3 Auto Drn Vlv Inlet Isol) (TB-594, 1H-32)
- 1SM-163 (Main Turb S/V #4 Auto Drn VIv Inlet Isol) (TB-594, 1H-32)
- 1SM-130 (SM Equalization Hdr Trap T-05 Inlet Isol) (TB-594, 1H-32)
- 1SM-137 (SM Equalization Hdr To Trap T-06 Inlet Isol) (TB-594, 1H-32)
- 1SB-32 (Main Stm Byp To Cond Hdr Stm Trap Inlet) (TB-594, 1G-29).

_ 3. Close 1SP-34 (SM To CFPT 1A & 1B) (TB-603, 1G-32).

_ 4. Notify control room personnel of status.

Enclosure 3 - Page 1 of 2 Local Operation of S/G PORVs

NOTE Emergency flashlights and Merlin-Gerins are available in the control room.

- 1. Obtain the following:
 - Flashlight
 - Merlin-Gerin.

NOTE The following are the preferred routes to the doghouses:

- Outside doghouse (1A and 1D S/G) Through southeast door of Unit 1 turbine building
- Inside doghouse (1B and 1C S/G) Through southeast control room exit to the auxiliary building.

2. Establish communications with the control room as follows:

- a. Obtain sound powered phones from storage on 594' elevation.
- Outside doghouse (DH-594, EE-44, Rm 591)
- Inside doghouse (DH-594, EE-52, Rm 572).
- b. Establish communications from the nearest phone jack at the selected S/G PORV(s):
 - Outside doghouse (DH-635, FF-43, Rm 591)
- Inside doghouse (DH-625, FF-53, Rm 572).

Enclosure 3 - Page 2 of 2 Local Operation of S/G PORVs

Severe damage to the actuator assembly can result if operated with the CAUTION clevis engaged to the actuator shaft and the equalizing valve closed. 1SV-1 and 1SV-19 turn counter clockwise to open. 1SV-7 and 1SV-13 turn NOTE clockwise to open. Place S/G PORV(s) in local operation as follows: З. a. Select desired PORV(s): • 1SV-19 (S/G 1A PORV Manual Ctrl) (DH-635, FF-GG, 43-44, Rm 591) 1SV-13 (S/G 1B PORV Manual Ctrl) (DH-635, FF, 53-54, Rm 572) • 1SV-7 (S/G 1C PORV Manual Ctrl) (DH-635, FF, 52-53, Rm 572) • 1SV-1 (S/G 1D PORV Manual Ctrl) (DH-635, FF-GG, 44-45, Rm 591). b. Unscrew clevis from manual override shaft. c. Turn handwheel in the "close" direction to expose actuator shaft below manual override shaft. d. Open equalizing valve on side of PORV actuator. ____e. Slide clevis onto actuator shaft. f. Turn handwheel to position valve as desired. WHEN directed by the control room, THEN restore S/G PORV(s) to control room control as 4. follows: a. Notify the control room to transmit a closed actuating signal from MCB to the valve positioner. ____b. Turn handwheel until the PORV is in the "closed" position and pressure is relieved from the clevis and actuator shaft. c. Remove clevis from actuator shaft. d. Turn handwheel until manual override shaft is fully extended. ____e. Screw clevis onto manual override shaft. _____f. Close equalizing valve on side of PORV actuator.

CNS EP/1/A/5000/E-3

Enclosure 4 - Page 1 of 5 Establishing Excess Letdown

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	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
1.	Align KC to excess letdown heat exchanger by opening the following valves:		
_	• 1KC-230A (Rx Bldg Non-Ess Hdr Isol)		
_	• 1KC-3A (Rx Bldg Non-Ess Ret Hdr Isol)		
	• 1KC-228B (Rx Bldg Non-Ess Hdr Isol)		
_	• 1KC-18B (Rx Bldg Non-Ess Ret Hdr Iso	I)	
_	 1KC-305B (Exs Letdn Hx Supply Cont Isol) 		
_	 1KC-315B (Exs Letdn Hx Ret Cont Isol) 		
2.	Establish NC pump seal return flow as follows:		
	a. Verify the following EMF trip 1 lights - DARK:		a. Perform the following:
	 1EMF-53A (Containment Trn A) 1EMF-53B (Containment Trn B). 		1) Notify station management to evaluate restoring NC pump seal return and excess letdown outside containment with high NC System activity.
			2) <u>WHEN</u> station management approval is obtained, <u>THEN</u> <u>RETURN</u> <u>TO</u> Step 2.b.
			3) GO TO Step 9.
<u> </u>	_ b. Verify NC pump seal injection flow - GREATER THAN 6 GPM TO EACH N	С	b. Perform the following:
	PUMP.	-	 <u>WHEN</u> NC pump seal injection is restored, <u>THEN</u> <u>RETURN</u> <u>TO</u> Step 2.b.
			2) GO TO Step 9.

CNS STEAM GENERATOR TUBE RUPTURE PAGE NO. EP/1/A/5000/E-3 57 of 73 Enclosure 4 - Page 2 of 5 **Revision 19** Establishing Excess Letdown ACTION/EXPECTED RESPONSE **RESPONSE NOT OBTAINED** 2. (Continued) c. Verify 1AD-7, D/1 "SEALWATER HX c. Perform the following: KC HI/LO FLOW" - DARK. 1) Notify station management to evaluate restoring NC pump seal return flow. 2) WHEN notified by station management OR 1AD-7, D/1 dark, THEN RETURN TO Step 2.d. 3) GO TO Step 9. d. Verify NCDT pressure - LESS THAN d. Perform the following: VCT PRESSURE. 1) WHEN NCDT pressure is less than VCT pressure, THEN RETURN TO Step 2.d. 2) **<u>GO</u><u>TO</u>** Step 9. e. Open the following valves: 1NV-89A (NC Pmps Seal Ret Cont Isol) 1NV-91B (NC Pmps Seal Ret Cont Isol). f. IF AT ANY TIME NCDT pressure is greater than VCT pressure, THEN perform the following: 1) Monitor NC Pump #1 seal ΔP . 2) Ensure the manual loader for 2) Verify excess letdown - ISOLATED. 1NV-124B (Excess Letdn Press Ctrl) is adjusted to 0%. Close the following valves: 1NV-89A (NC Pmps Seal Ret Cont Isol) 1NV-91B (NC Pmps Seal Ret Cont Isol). 4) GO TO Step 9.

STEAM GENERATOR TUBE RUPTURE

Enclosure 4 - Page 3 of 5 Establishing Excess Letdown

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I			1
	ACTION/EXPECTED RESPONSE	 RESPONSE NOT OBTAINED	l
3.	 Notify Primary Chemistry of the following: Excess Letdown will be placed in service. VCT pressure will be reduced to 25 PSIG. 		
4.	Dispatch operator to adjust the VCT hydrogen pressure to 25 PSIG. Refer to OP/1/A/6200/001 (Chemical and Volume Control System).		
5.	Ensure the manual loader for 1NV-124E (Excess Letdn Press Ctrl) is adjusted to 0%.		
6.	Verify the following conditions:	Perform the following:	
	 VCT pressure - BETWEEN 25 PSIG AND 30 PSIG. 	 a. Place 1NV-125B (Excess Letdn Hx Otlt Ctrl) in the "NCDT" position.	
	 The following valves - OPEN: 	 b. <u>WHEN</u> conditions met, <u>THEN RETURN</u> <u>TO</u> Step 5.	
	 1NV-188A (VCT Otil Isol) 1NV-189B (VCT Otil Isol). 	 c. <u>GO TO</u> Step 9.	
7.	Place 1NV-125B (Excess Letdn Hx Otit Ctrl) in the "VCT" position.		
8.	<u>GO TO</u> Step 14.		
9.	Verify the following EMF trip 1 lights - DARK:	 <u>GO TO</u> Step 13.	
-	 1EMF-53A (Containment Trn A) 1EMF-53B (Containment Trn B). 		

STEAM GENERATOR TUBE RUPTURE

Enclosure 4 - Page 4 of 5 Establishing Excess Letdown

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г					
	ACTION/EXPECTED RESPONSE		RESPONSE	NOT OBTAINED	
10.	Align KC to the NCDT heat exchanger b opening the following valves:	уy			
_	 1KC-320A (NCDT Hx Cool Supply Cont Isol) 				
_	• 1KC-333A (NCDT Hx Cool Ret Cont Iso	d)			
_	 1KC-332B (NCDT Hx Cool Ret Cont Isol). 				
11.	Ensure at least one NCDT pump - ON.				
12.	Open the following valves:				
_	 1WL-805A (NCDT Pump Disch Cont Iso 1WL-807B (NCDT Pumps Disch Cont Isol). 	ol)			
13.	Place 1NV-125B (Excess Letdn HX Otlt Ctrl) in the "NCDT" position.				
14.	Open the following valves:				
	• 1NV-122B (Loop C To Exs Letdn Hx Isc	ol)			
	 1NV-123B (Loop C To Exs Letdn Hx Isol). 				
15.	Select "OPEN" on 1NV-124B.				
16.	Slowly adjust the manual loader for 1NV-124B to 6% open.				

STEAM GENERATOR TUBE RUPTURE

Enclosure 4 - Page 5 of 5 Establishing Excess Letdown

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ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

Manually throttle 1NV-124B until the
required excess letdown flow is achieved
and within the following parameters:

- Excess letdown Hx outlet temperature -LESS THAN 170°F
- Excess letdown Hx outlet pressure LESS THAN OR EQUAL TO 45 PSIG.

STEAM GENERATOR TUBE RUPTURE

Enclosure 5 - Page 1 of 3 NC Pressure And Makeup Control to Minimize Leakage PAGE NO. 61 of 73 Revision 19

	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
1.	Depressurize NC System to prevent ruptured S/G(s) PORV from lifting as follows:	
	_ a. Verify NC pressure - LESS THAN 1125 PSIG.	 a. Perform the following: 1) IF normal Pzr spray flow is available, <u>THEN</u> depressurize NC System to less than 1125 PSIG using normal Pzr spray. 2) IF normal Pzr spray is not available, <u>THEN</u> perform the following: a) IF letdown is in service, <u>THEN</u> establish NV aux spray as follows: (1) Ensure the following valves - CLOSED: • 1NC-27 (Pzr Spray Ctrl Frm Loop A) • 1NC-29 (Pzr Spray Ctrl Frm Loop B) • 1NV-39A (NV Supply To Loop D Isol) • 1NV-32B (NV Supply To Loop A Isol). (2) Maintain charging flow less than 180 GPM. (3) Throttle 1NV-37A (NV Supply To Pzr Aux Spray) and charging flow to depressurize NC System to less than 1125 PSIG. b) IF letdown is not in service OR NV aux spray is not available, THEN use one Pzr PORV to depressurize NC System to less than 1125 PSIG.
-	b. Maintain NC System pressure less the 1125 PSIG.	an

STEAM GENERATOR TUBE RUPTURE

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Enclosure 5 - Page 2 of 3 NC Pressure And Makeup Control to Minimize Leakage

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 2. Control NC pressure and charging flow to minimize primary to secondary leakage as follows:
 - ____a. Perform appropriate action(s) from the table below to equalize NC and ruptured S/G pressure:

	HIGHEST RUPTURED S/G N/R LEVEL		
PZR LEVEL	INCREASING	DECREASING	OFFSCALE HIGH
LESS THAN 25% (34% ACC)	 Increase charging flow Depressurize NC System using Step 2.b 	Increase charging flow	 Increase charging flow Maintain NC and ruptured S/G(s) pressures equal
BETWEEN 25% (34% ACC) AND 50%	Depressurize NC System using Step 2.b	Turn on Pzr heaters	Maintain NC and ruptured S/G(s) pressures equal
BETWEEN 50% AND 76% (73% ACC)	 Depressurize NC System using Step 2.b Decrease charging flow 	Turn on Pzr heaters	Maintain NC and ruptured S/G(s) pressures equal
GREATER THAN 76%(73% ACC)	Decrease charging flow	Turn on Pzr heaters	Maintain NC and ruptured S/G(s) pressures equal

STEAM GENERATOR TUBE RUPTURE

Enclosure 5 - Page 3 of 3 NC Pressure And Makeup Control to Minimize Leakage

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	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2	. (Continued) b. Depressurize NC System as required b table above as follows:	у.
	1) Verify normal Pzr spray flow - AVAILABLE.	 Perform the following: IF letdown is in service, THEN establish NV aux spray as follows:
	2) Use normal Pzr spray to depressurize NC System.	

STEAM GENERATOR TUBE RUPTURE

Enclosure 6 - Page 1 of 2 Auxiliary System Alignment

1. Transfer turbine steam seal supply to AS as follows:
a. Open 1TL-8 (Aux Stm To Stm Seal Reg).
b. Close 1TL-2 (Main Stm To Stm Seal Reg).
2. Align AEB feedwater supply and recirc to Unit 2. <u>REFER TO</u> OP/1/B/6250/007B (Auxiliary Electric Boilers).
3. IF Unit 2 is available to supply Unit 1 AS, <u>THEN</u> dispatch operator to align Unit 2 AS as follows:
a. Ensure 1AS-33 (Unit 1 AS Hdr Isol) (TB-590, 1M-26) - OPEN.
b. Open 1AS-59 (Unit 2 AS Hdr Isol) (TB-584, 2N-26).
 4. <u>IF</u> Unit 2 is not available to supply Unit 1 AS, <u>THEN</u> align AEB to Unit 1 AS. <u>REFER TO</u> OP/1/B/6250/007B (Auxiliary Electric Boilers).
5. Dispatch operator to align NB and WL evaporator condensate to Unit 2 CST as follows:
a. Open 1CS-118 (NB & WL Waste Evap Cond Ret To Unit 2 CST) (AB-553, JJ-53, Rm 217) (Ladder needed).
b. Close 1CS-117 (NB & WL Waste Evap Cond Ret To Unit 1 CST) (AB-553, JJ-53, Rm 217).
6. Dispatch operator to align CSAEs to the AS header as follows:
a. Ensure 1SA-27 (Aux Stm To CSAE) (TB-594, 1M-27) - OPEN.
b. Close 1SA-22 (Main Stm To CSAE) (TB-594, 1M-32).
7. Align AS to CFPTs as follows:
a. Ensure 1AS-12 (AS To CFPT Isol) (TB-605, 1M-27) - OPEN.
b. Dispatch operator to ensure 1SP-3 (SC To CFPT 1A & 1B) (TB-640, 1G-24) - CLOSED.

STEAM GENERATOR TUBE RUPTURE

Enclosure 6 - Page 2 of 2 Auxiliary System Alignment

8. <u>IF</u> Unit 2 condensate is available to supply CA storage tank <u>AND</u> station management approves, <u>THEN</u> dispatch operator to perform the following:

- ____a. Ensure 2CM-383 (CA CST Inlet) (TB-614, 2K-22) OPEN.
- ____b. Close 1CM-383 (CA CST Inlet) (TB-614, 1K-22).
- ____ c. Ensure 1CS-74 (CA CST Drn To Unit 2 CST) (SB-619, T-25) OPEN.
- _____d. Close 1CS-73 (CA CST Drn To Unit 1 CST) (SB-619, T-25).
- 9. <u>IF</u> Unit 1 CST overflow is imminent <u>OR</u> Unit 2 CST has low level, <u>THEN</u> coordinate with RP to pump to Unit 2. <u>REFER TO</u> OP/1/A/6250/001 (Condensate and Feedwater System).
- 10. Coordinate with RP and determine if the WP System should be aligned to receive contaminated drains. <u>REFER TO PT/1/B/4150/001G</u> (Turbine Building Sump Isolation).
- 11. <u>IF WP sump is contaminated, THEN</u> transfer sump to Monitor Tank Bldg. <u>REFER TO</u> OP/1/B/6500/013 (Turbine Building Sump System).

STEAM GENERATOR TUBE RUPTURE

Enclosure 7 - Page 1 of 7 NC Pump Start PAGE NO. 66 of 73 Revision 19

RESPONSE NOT OBTAINED ACTION/EXPECTED RESPONSE Perform the following: Verify NC pump seal cooling has been 1. maintained from one of the following: _ a. IF NC pump seal cooling has not been maintained, THEN notify station · Seal injection flow management to perform a status evaluation prior to starting an NC pump. OR b. WHEN the status evaluation has been • KC flow to thermal barrier. completed, THEN GO TO Step 2. c. Do not continue in this enclosure. Verify Pzr level - LESS THAN OR EQUAL 2. IF the NC System is intact, THEN ensure TO 92%. a steam bubble exists in the Pzr prior to starting an NC pump. Place normal Pzr spray valves in manual З. and closed. **Ensure Phase B Containment Isolation -**4. RESET.

STEAM GENERATOR TUBE RUPTURE

Enclosure 7 - Page 2 of 7 NC Pump Start

ACTION/EXPECTED RESPONSE

- 5. Verify all of the following KC valves -OPEN:
 - 1KC-230A (Rx Bldg Non-Ess Hdr Isol)
 - 1KC-3A (Rx Bldg Non-Ess Ret Hdr Isol)
 - 1KC-394A (NC Pump 1A Therm Bar Otlt)
 - 1KC-425A (NC Pumps Ret Hdr Cont Isol)
 - 1KC-345A (NC Pump 1C Therm Bar Otlt)
 - 1KC-228B (Rx Bldg Non-Ess Hdr Isol)
 - 1KC-18B (Rx Bldg Non-Ess Ret Hdr Isol)
 - 1KC-364B (NC Pump 1B Therm Bar Otlt)
 - 1KC-338B (NC Pumps Sup Hdr Cont Isol)
 - 1KC-424B (NC Pumps Ret Hdr Cont Isol)
 - 1KC-413B (NC Pump 1D Therm Bar Otlt).

6. Establish NC pump motor cooling as follows:

- a. Ensure the following valves OPEN:
- 1RN-484A (Lower Cont Vent Unit Ret)
- 1RN-487B (Lower Cont Vent Unit Ret)
- 1RN-437B (Lower Cont Vent Unit Sup).

RESPONSE NOT OBTAINED

Perform the following:

- ____a. Manually open affected valve(s).
- b. <u>IF</u> KC cooling cannot be established from one operating KC train to the NC pumps, <u>THEN</u> return this enclosure to the Control Room SRO.

STEAM GENERATOR TUBE RUPTURE

Enclosure 7 - Page 3 of 7 NC Pump Start PAGE NO. 68 of 73 Revision 19

ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED				
6. (Continued)					
b. Verify "YV OPERABLE" light (rear of 1MC-7) - LIT.	 b. Perform the following: 1) Dispatch operator to restore YV System to normal. <u>REFER TO</u> OP/1/A/6450/020 (Containment Chill Water System). 2) <u>IF</u> YV System cannot be restored to normal, <u>THEN</u>: 				
	 a) Select "RN" on "YV/RN COOL WATER MODE". b) Ensure the following valves - OPEN: 				
	 1RN-49A (Non-Ess Supply Hdr Isol) 1RN-50B (Non-Ess Supply Hdr Isol) 				
· · ·	 1RN-51A (Non-Ess Ret Hdr Isol) 1RN-52B (Non-Ess Ret Hdr Isol). c) Ensure at least one of the following valves - OPEN: 				
	 1RN-48B (RN Supply X-Over Isol) 1RN-47A (RN Supply X-Over Isol). 				
	3) IF NC pump motor cooling cannot be established, <u>THEN</u> return this enclosure to the Control Room SRO.				

STEAM GENERATOR TUBE RUPTURE

Enclosure 7 - Page 5 of 7 NC Pump Start

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ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINE	D
7. (Continued)			
 KC flow to NC pump motor lower bearin between 4 GPM and 9.5 GPM by one o the following methods: 			
 NC pump motor lower bearing KC outlet Hi/Lo flow alarm (1AD-6) - DARK 			
OR			
 OAC NC pump graphic 			
OR			
 Dispatch operator to locally verify adequate KC cooling to NC pump motor lower bearing (AB-543, FF-54, Rm 200). 	,		
 NC pump upper/lower oil reservoir Lo level alarm (1AD-6) - DARK 			
 NC pump seal injection flow - GREATE THAN 6 GPM 	R		
 NC pump seal leakoff flow - WITHIN LIMITS OF UNIT 1 REVISED DATA BOOK FIGURE 26. 			
 1AD-11, K/6 "230 KV SWITCHYARD VOLTAGE LO" - DARK. 			

STEAM GENERATOR TUBE RUPTURE

Enclosure 7 - Page 6 of 7 NC Pump Start

	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
8.	Verify "REACTOR VESSEL UR LEVEL" GREATER THAN OR EQUAL TO 100%.		IF at least one NCP is available to start, THEN perform the following:
			a. Increase Pzr level to greater than 90%.
			 Increase NC subcooling based on core exit T/Cs to greater than 36°F.
		_	 Operate Pzr heaters as necessary to maintain Pzr saturated.
			d. <u>WHEN</u> the following conditions met, <u>THEN GO TO</u> Step 9.
			• Pzr level - GREATER THAN 90%.
			 NC subcooling based on core exit T/Cs - GREATER THAN 36°F.
			• Pzr - SATURATED
		—	e. Do not continue in this enclosure.
9.	Start one NC pump oil lift pump on the NC pump to be started.	9	
10.	<u>WHEN</u> 2 minutes has elapsed, <u>THEN</u> start the NC pump.		
11.	<u>WHEN</u> 3 minutes has elapsed after NC pump reaches full speed, <u>THEN</u> ensure the NC pump oil lift pump stops.		

STEAM GENERATOR TUBE RUPTURE

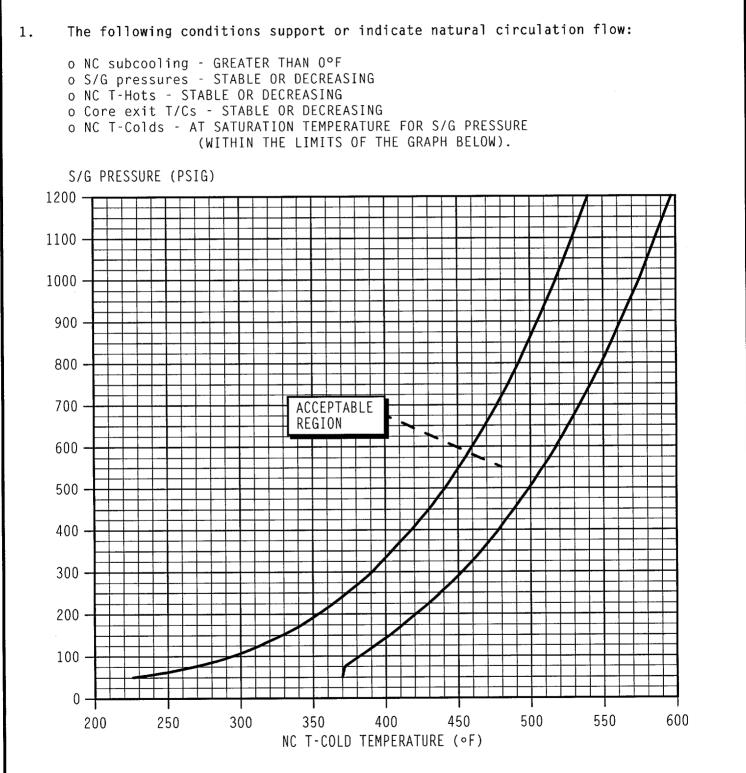
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RESPONSE NOT OBTAINED ACTION/EXPECTED RESPONSE Verify the following conditions satisfied Perform the following: 12. for the NC pump that was started: a. Stop the affected NC pump. • #1 seal △P - GREATER THAN OR b. Return this enclosure to the Control EQUAL TO 200 PSID Room SRO. • #1 seal outlet temperature - LESS THAN 235°F • NC pump lower bearing temperature -LESS THAN 225°F NC pump seal injection flow - GREATER THAN 6 GPM NC pump seal leakoff flow - WITHIN LIMITS OF UNIT 1 REVISED DATA **BOOK FIGURE 26.** NC pump shaft vibration - LESS THAN 20 MILS • NC pump motor frame vibration - LESS THAN 5 MILS • IF OAC is available, THEN verify the following: Stator winding temperature - LESS THAN 311°F • Motor bearing temperature - LESS THAN 195°F.

STEAM GENERATOR TUBE RUPTURE

Enclosure 8 - Page 1 of 1 Natural Circulation Monitoring Parameters

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2. <u>IF</u> Natural Circulation flow is not established, <u>THEN</u> increase dumping steam to establish Natural Circulation flow.