

June 22, 19

Docket No. 50-333

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Mr. John C. Brons
 Executive Vice President - Nuclear Generation
 Power Authority of the State of New York
 123 Main Street
 White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 72827)

The Commission has issued the enclosed Amendment No. 132 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 22, 1989.

The amendment clarifies the surveillance requirements for maintaining the Emergency Core Cooling Systems and the Reactor Core Isolation Cooling System discharge piping filled.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

David E. LaBarge, Project Manager
 Project Directorate I-1
 Division of Reactor Projects, I/II

Enclosures:

1. Amendment No.132 to DPR-59
2. Safety Evaluation

cc: w/enclosures
 See next page

OF01
 1/1

[AMEND 333 TAC 72827]

*See previous concurrence

OFC	:PDI-1	:PDI-1	:SPLB*	:OGC*	:PDI-1	:	:
NAME	:CVogan	:DLaBarge	:vr	:JCraig	:RCapra	:	:
DATE	:6/5/89	:6/22/89	:5/18/89	:6/15/89	:6/22/89	:	:

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 PDC

Mr. John C. Brons
Power Authority of the State of New York

James A. Fitzpatrick Nuclear
Power Plant

cc:

Mr. Gerald C. Goldstein
Assistant General Counsel
Power Authority of the State
of New York
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Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 132
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated March 22, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 132, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Page

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Insert Page

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3.5 (cont'd)

condition, that pump shall be considered inoperable for purposes of satisfying Specifications 3.5.A, 3.5.C, and 3.5.E.

H. Average Planar Linear Heat Generation Rate (APLHGR)

During power operation, the APLHGR for each type of fuel as a function of axial location and average planar exposure shall be within limits based on applicable APLHGR limit values which have been approved for the respective fuel and lattice types. When hand calculations are required, the APLHGR for each type of fuel as a function of average planar exposure shall not exceed the limiting value for the most limiting lattice (excluding natural uranium) shown in Figures 3.5-11 through 3.5-14 during two recirculation loop operation. During single loop operation, the APLHGR for each fuel type shall not exceed the above values multiplied by 0.84 (see Bases 3.5.K, Reference 1). If at anytime during reactor power operation greater than 25% of rated power it is determined that the limiting value for APLHGR is being exceeded, action shall then be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, an orderly reactor power reduction shall be commenced immediately. The reactor power shall be reduced to less than 25% of rated power within the next four hours, or until the APLHGR is returned to within the prescribed limits.

4.5 (cont'd)

2. Following any period when the LPCI subsystems or core spray subsystems have not been maintained in a filled condition; the discharge piping of the affected subsystem shall be vented from the high point of the system and water flow observed.
3. Whenever the HPCI or RCIC System is lined up to take suction from the condensate storage tank, the discharge piping of the HPCI or RCIC shall be vented from the high point of the system, and water flow observed on a monthly basis.
4. The level switches located on the Core Spray and RHR System discharge piping high points which monitor these lines to insure they are full shall be functionally tested each month.

H. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.



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NUCLEAR REGULATORY COMMISSION
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POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

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License No. DPR-59

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 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 132, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Page

123

Insert Page

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3.5 (cont'd)

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During power operation, the APLHGR for each type of fuel as a function of axial location and average planar exposure shall be within limits based on applicable APLHGR limit values which have been approved for the respective fuel and lattice types. When hand calculations are required, the APLHGR for each type of fuel as a function of average planar exposure shall not exceed the limiting value for the most limiting lattice (excluding natural uranium) shown in Figures 3.5-11 through 3.5-14 during two recirculation loop operation. During single loop operation, the APLHGR for each fuel type shall not exceed the above values multiplied by 0.84 (see Bases 3.5.K, Reference 1). If at anytime during reactor power operation greater than 25% of rated power it is determined that the limiting value for APLHGR is being exceeded, action shall then be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, an orderly reactor power reduction shall be commenced immediately. The reactor power shall be reduced to less than 25% of rated power within the next four hours, or until the APLHGR is returned to within the prescribed limits.

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4. The level switches located on the Core Spray and RHR System discharge piping high points which monitor these lines to insure they are full shall be functionally tested each month.

H. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 132 TO FACILITY OPERATING LICENSE NO. DPR-59
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

INTRODUCTION

By letter dated March 22, 1989, the Power Authority of the State of New York (PASNY or the licensee), requested changes to the Technical Specifications (TS) for the James A. FitzPatrick Nuclear Power Plant. The changes would clarify the surveillance requirements for maintaining the discharge piping of the Emergency Core Cooling Systems (ECCS) and of the Reactor Core Isolation Cooling System (RCICS) filled with water whenever they are required to be operable. Another minor editorial change is also included.

DESCRIPTION

Since the ECCS and the RCICS are designed for automatic and rapid startup under all conditions when they must be considered operable, their discharge piping must be kept filled with water to prevent water hammer damage should startup occur. A supply of water at low pressure is connected to each system and periodic, manual venting from the high point of each system ensures that the piping is full of water. Present TS specifies that the discharge piping of the Core Spray System (CSS) and the Low Pressure Coolant Injection System (LPCIS) - which are a part of the ECCS - must be vented prior to the monthly system operability test and following any period during which the respective systems have been declared inoperable prior to declaring them operable.

However, these systems may be declared inoperable as a result of conditions which do not involve a loss of water in the discharge piping (e.g., instrumentation problems, valve operator problems). For these conditions, venting is not needed in order to ensure that the discharge piping is full of water. Therefore, to more clearly address the venting need and ensure that venting occurs when appropriate, this TS proposal would revise the surveillance requirement such that venting and filling operations must be conducted following any period when the piping has not been maintained in a filled condition, rather than following any period of system inoperability. Thus, venting is related to a loss of water condition and the monthly system operability test, rather than any other condition and is considered acceptable. This change affects TS Section 4.5.G.2.

Another proposed change would delete the requirement to vent the CSS discharge piping from the high point whenever the CSS is lined up to take a suction from the Condensate Storage Tank (CST). This evolution is presently required by Section 4.5.G.3 on a monthly basis.

The normal CSS suction lineup is from the suppression chamber. To transfer the lineup to the CST requires shutting the suppression chamber key-locked motor operated valve and opening the manual valve to the CST. The licensee has stated that, if the lineup is shifted to the CST, the CSS would not be considered operable and that venting would be addressed by procedures which would be developed or in place at the time. The staff agrees that this is appropriate.

Also, this monthly venting requirement is redundant to the existing monthly venting surveillance requirement in Section 4.5.G.1, which states that the CSS must be vented prior to the monthly operability test. Furthermore, since this amendment will require that the CSS be vented following any period when it was not maintained in a filled condition, this requirement will apply before the system is returned to service, regardless of the suction flow path. Therefore, the staff agrees that proper requirements remain in place which will protect the ECCS and RCIC discharge piping from water hammer damage.

An editorial change to Specification 3.5.G.a would add "of" between "purposes" and "satisfying" to clarify the statement.

The proposed changes clarify the intent of the Technical Specifications, improve the consistency within the Technical Specifications, and correct an editorial error. The proposed changes do not involve modification of any existing equipment, systems, or components; nor do they change any administrative controls or limitations imposed on existing plant equipment. The changes do not alter the conclusions of the plant's accident analyses or radiological release analyses as documented in the Final Safety Analysis Report or Safety Evaluation Report. They are, therefore, acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Part 51.22(c)(9). Pursuant to 10 CFR Part 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 22, 1989

PRINCIPAL CONTRIBUTOR:

D. E. LaBarge



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
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