

September 5, 1989

Docket No. 50-333

Mr. John C. Brons  
Executive Vice President - Nuclear Generation  
Power Authority of the State of New York  
123 Main Street  
White Plains, New York 10601

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Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 73290)

The Commission has issued the enclosed Amendment No. 136 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 19, 1989.

The amendment corrects certain errors in Tables 4.1-1, 4.1-2 and 4.2-1.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

David E. LaBarge, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 136 to DPR-59
2. Safety Evaluation

cc: w/enclosures  
See next page

[AMEND 73290]

*concur subject to change. JM 8/15/89*

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in dark ink, appearing to read "D. E. LaBarge", is written over the typed name.

David E. LaBarge, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 136 to DPR-59
2. Safety Evaluation

cc: w/enclosures  
See next page

Mr. John C. Brons  
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James A. FitzPatrick Nuclear  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 136  
License No. DPR-59


1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated May 19, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 136, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 5, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 136

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

45  
46  
47  
78

Insert Pages

45  
46  
47  
78

Table 4.1-1 (cont'd)

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT FUNCTIONAL TEST  
MINIMUM FUNCTIONAL TEST FREQUENCIES FOR SAFETY INSTRUMENT AND CONTROL CIRCUITS

<u>Instrument Channel</u>	<u>Group (2)</u>	<u>Functional Test</u>	<u>Minimum Frequency (3)</u>
Main Steam Line High Radiation	B	Trip Channel and Alarm (4)	Once/week.
Main Steam Line Isolation Valve Closure	A	Trip Channel and Alarm	Once/month. (1)
Turbine Control Valve EHC Oil Pressure	A	Trip Channel and Alarm	Once/month.
Turbine First Stage Pressure Permissive	B	Trip Channel and Alarm (4)	Once/month. (1)(8)
Turbine Stop Valve Closure	A	Trip Channel and Alarm	Once/month. (1)

NOTES FOR TABLE 4.1-1

- Initially once every month until acceptable failure rate data are available; thereafter, a request may be made to the NRC to change the test frequency. The compilation of instrument failure rate data may include data obtained from other boiling water reactors for which the same design instrument operates in an environment similar to that of JAFNPP.
- A description of the three groups is included in the Bases of this Specification.
- Functional tests are not required on the part of the system that is not required to be operable or are tripped.  
If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.
- This instrumentation is exempted from the instrument channel test definition. This instrument channel functional test will consist of injecting a simulated electrical signal into the instrument channels.

Table 4.1-2

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT CALIBRATION  
MINIMUM CALIBRATION FREQUENCIES FOR REACTOR PROTECTION INSTRUMENT CHANNELS

<u>Instrument Channel</u>	<u>Group (1)</u>	<u>Calibration (4)</u>	<u>Minimum Frequency (2)</u>
IRM High Flux	C	Comparison to APRM on Controlled Shutdowns	Maximum frequency once/week
APRM High Flux Output Signal	B	Heat Balance	Daily
Flow Bias Signal	B	Internal Power and Flow Test with Standard Pressure Source	Every refueling outage
LPRM Signal	B	TIP System Traverse	Every 1000 effective full power hours
High Reactor Pressure	B	Standard Pressure Source	Note (7)
High Drywell Pressure	B	Standard Pressure Source	Note (7)
Reactor Low Water Level	B	Standard Pressure Source	Note (7)
High Water Level in Scram Discharge Instrument Volume	A	Water Column, Note (6)	Once/operating cycle, Note (6)
High Water Level in Scram Discharge Instrument Volume	B	Standard Pressure Source	Every 3 months
Main Steam Line Isolation Valve Closure	A	Note (5)	Note (5)
Main Steam Line High Radiation	B	Standard Current Source (3)	Every 3 months
Turbine First Stage Pressure Permissive	B	Standard Pressure Source	Note (7)



Table 4.1-2 (cont'd)

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT CALIBRATION  
MINIMUM CALIBRATION FREQUENCIES FOR REACTOR PROTECTION INSTRUMENT CHANNELS

<u>Instrument Channel</u>	<u>Group (1)</u>	<u>Calibration (4)</u>	<u>Minimum Frequency (2)</u>
Turbine Control Valve Fast Closure Oil Pressure Trip	A	Standard Pressure Source	Once/operating cycle
Turbine Stop Valve Closure	A	Note (5)	Note (5)

NOTES FOR TABLE 4.1-2

1. A description of three groups is included in the Bases of this Specification.
2. Calibration test is not required on the part of the system that is not required to be operable, or is tripped, but is required prior to return to service.
3. The current source provides an instrument channel alignment. Calibration using a radiation source shall be made each refueling outage.
4. Response time is not a part of the routine instrument channel test but will be checked once per operating cycle.
5. Actuation of these switches by normal means will be performed during the refueling outages.
6. Calibration shall be performed utilizing a water column or similar device to provide assurance that damage to a float or other portions of the float assembly will be detected.
7. Sensor calibration once per operating cycle. Master/slave trip unit calibration once per 6 months.

TABLE 4.2-1  
MINIMUM TEST AND CALIBRATION FREQUENCY FOR PCIS

<u>Instrument Channel (8)</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check(4)</u>
1) Reactor High Pressure (Shutdown Cooling Permissive)	(1)	Once/3 months	None
2) Reactor Low-Low-Low Water Level	(1)(5)	(15)	Once/day
3) Main Steam High Temp.	(1)(5)	(15)	Once/day
4) Main Steam High Flow	(1)(5)	(15)	Once/day
5) Main Steam Low Pressure	(1)(5)	(15)	Once/day
6) Reactor Water Cleanup High Temp.	(1)	Once/3 months	None
7) Condenser Low Vacuum	(1)(5)	(15)	Once/day

<u>Logic System Functional Test (7) (9)</u>	<u>Frequency</u>
1) Main Steam Line Isolation valves Main Steam Line Drain Valves Reactor Water Sample Valves	Once/6 months
2) RHR - Isolation Valve Control Shutdown Cooling Valves Head Spray	Once/6 months
3) Reactor Water Cleanup Isolation	Once/6 months
4) Drywell Isolation Valves Tip Withdrawal Atmospheric Control Valves	Once/6 months
5) Standby Gas Treatment System Reactor Building Isolation	Once/6 months

NOTE: See listing of notes following Table 4.2-6 for the notes referred to herein.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 136 TO FACILITY OPERATING LICENSE NO. DPR-59  
POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
DOCKET NO. 50-333

INTRODUCTION

By letter dated May 19, 1989, the Power Authority of the State of New York (PASNY or the licensee) requested changes to the Technical Specification (TS) for the James. A. FitzPatrick Nuclear Power Plant. The changes would revise Tables 4.1-1, 4.1-2 and 4.2-1 to correct certain errors which were introduced by previous TS amendments and to correct an editorial error in Table 4.1-2.

DESCRIPTION

The proposed change to Table 4.1-1, "Reactor Protection System (Scram) Instrument Functional Test," and Table 4.1-2, "Reactor Protection System (Scram) Instrument Calibration," would remove the testing requirements associated with the reactor pressure permissive switches which establish the pressure setpoint below which the main steam line isolation valve (MSIV) reactor scram signal and the MSIV isolation signal on low condenser vacuum signal are bypassed when the mode switch is in the refuel or startup positions. Removal of the pressure switches (designated 02-3PS-51 A through D) from the scram and the Primary Containment Isolation System (PCIS) logic requirements of TS Table 3.1-1, "Reactor Protection System (Scram) Instrumentation Requirements," and Table 3.2-1, "Instrumentation that Initiates Primary Containment Isolation," was approved in Amendment No. 122, dated February 7, 1989. However, due to an oversight, the submittal for this amendment did not address the related instrument tests designated in Tables 4.1-1 and 4.1-2. Since the safety analysis concerning removal of the switches has already been performed in Amendment No. 122, with the determination that there is no adverse impact on safety, removal of the testing requirements is equally justified and is approved.

Another change to Table 4.1-2 would correct a typographical error introduced in Amendment No. 89 by changing "Trip System Traverse" to "TIP System Traverse." All previous amendments which affected this table reflect use of the abbreviation "TIP" and Amendment No. 89 does not address a change to this terminology. In addition, to perform an LPRM Signal Instrument channel Calibration, a Traversing Incore Probe (TIP) System traverse method is used, as described in the Final Safety Analysis Report. Also, the change would make the table consistent with the description in the TS Bases. For these reasons, the change is determined to be editorial in nature and is approved.

The proposed change to Table 4.2-1, "Minimum Test and Calibration Frequency for PCIS," would change the water level instrument designation from Low-Low to Low-Low-Low. This instrumentation causes the main steam isolation valves, the main steam line drain valves, the reactor water sample valves, and the standby gas treatment system reactor building isolations valves, to shut automatically. The actual change to the setpoint was issued as Amendment No. 103, dated December 19, 1986. However, due to an oversight, the submittal for this amendment did not address the related instrument tests designated in Table 4.2-1. Since the safety analysis concerning the setpoint change has already been performed in Amendment No. 103, with the determination that there is no adverse impact on safety, the change to the table is considered to be editorial and will result in improved TS consistency. It is, therefore, approved.

#### ENVIRONMENTAL CONSIDERATION

This amendment involves a change to surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Part 51.22(c)(9). Pursuant to 10 CFR Part 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 5, 1989

#### PRINCIPAL CONTRIBUTOR:

D. LaBarge