

Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616-465-5901



March 16, 2001

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-58 and DPR-74
Docket Nos. 50-315 and 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following revised report is being submitted:

LER 315/1998-020-02: "Retraction of LER 315/1998-020-00"

This LER supplement is being submitted to retract LER 315/1998-020-00.

No new commitments were identified in this submittal.

Should you have any questions regarding this correspondence, please contact Mr. Ronald W. Gaston, Manager, Regulatory Affairs, at 616/465-5901, extension 1366.

Sincerely,

Joseph E. Pollock
Plant Manager

A handwritten signature in black ink that reads 'Joseph E. Pollock'. The signature is written over the printed name and title. There is a small circle around the initials 'JEP'.

Attachment

c: J. E. Dyer, Region III
A. C. Bakken
L. Brandon
T. P. Noonan
R. P. Powers
M. W. Rencheck
R. Whale
NRC Resident Inspector
Records Center, INPO

IE22

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (T-6 F33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1)

Donald C. Cook Nuclear Plant Unit 1

DOCKET NUMBER (2)

05000-315

PAGE (3)

1 OF 2

TITLE (4)

Retraction of LER 315/1998-020-00

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAME | DOCKET NUMBER |
| 04 | 08 | 1998 | 1998 | 020 | 02 | 3 | 16 | 2001 | Cook Nuclear Plant Unit 2 | 05000-316 |
| | | | | | | | | | FACILITY NAME | DOCKET NUMBER |

| OPERATING MODE (9) | POWER LEVEL (10) | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11) | | | | | | | | | | | | | | | | | | | | | |
|--------------------|------------------|---|--|---|--|---|--|---|--|---|--|---|--|---|--|---|--|---|--|--|---|--------------------------------|--|
| — | 00 | <input type="checkbox"/> 20.2201(b) | <input type="checkbox"/> 20.2203(a)(1) | <input type="checkbox"/> 20.2203(a)(2)(i) | <input type="checkbox"/> 20.2203(a)(2)(ii) | <input type="checkbox"/> 20.2203(a)(2)(iii) | <input type="checkbox"/> 20.2203(a)(2)(iv) | <input type="checkbox"/> 20.2203(a)(3)(i) | <input type="checkbox"/> 20.2203(a)(3)(ii) | <input type="checkbox"/> 20.2203(a)(3)(iii) | <input type="checkbox"/> 20.2203(a)(4) | <input type="checkbox"/> 50.73(a)(2)(i) | <input type="checkbox"/> 50.73(a)(2)(ii) | <input type="checkbox"/> 50.73(a)(2)(iii) | <input type="checkbox"/> 50.73(a)(2)(iv) | <input type="checkbox"/> 50.73(a)(2)(v) | <input type="checkbox"/> 50.73(a)(2)(vi) | <input type="checkbox"/> 50.73(a)(2)(vii) | <input type="checkbox"/> 50.73(a)(2)(viii) | <input type="checkbox"/> 50.73(a)(2)(ix) | <input type="checkbox"/> 50.73(a)(2)(x) | <input type="checkbox"/> 73.71 | <input checked="" type="checkbox"/> OTHER Retraction |
| | | | | | | | | | | | | | | | | | | | | | | | Specify in Abstract below or in NRC Form 366A |

LICENSEE CONTACT FOR THIS LER (12)

NAME Richard Meister, Regulatory Compliance

TELEPHONE NUMBER (Include Area Code) 616/465-5901, x1707

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO EPIX | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO EPIX |
|-------|--------|-----------|--------------|--------------------|-------|--------|-----------|--------------|--------------------|
| | | | | | | | | | |
| | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| YES (If yes, complete EXPECTED SUBMISSION DATE). | X | NO | EXPECTED | MONTH | DAY | YEAR |
|--|---|----|----------|-------|-----|------|
| | X | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 8, 1998, with Units 1 and 2 in Cold Shutdown, as a result of questions raised by personnel performing a safety system functional inspection of the Containment Spray system, it was initially believed that the Westinghouse BORDER code used to determine post-accident pH in the containment sump had not included the sodium tetraborate contained within the ice condenser ice bed. The failure to include the sodium tetraborate in the calculation could have potentially resulted in the containment sump pH value exceeding the maximum allowable pH limit following a Loss of Coolant Accident. An ENS notification was conservatively made at 1440 hours EDT in accordance with 10CFR50.72(b)(2)(i), for an unanalyzed condition discovered while the plant was shutdown. Interim LERs 315/98-020-00, and -01 were submitted in accordance with 10CFR50.72(a)(2)(i).

This supplement replaces all previous revisions in their entirety. Note that the earlier versions incorrectly identified the chemical in the ice as sodium hydroxide instead of sodium tetraborate.

Subsequent investigation determined that the sump pH value would not have exceeded post LOCA pH limits assumed in the accident analysis.

The LER 315/98-020 series is hereby retracted as no violation of T/S occurred or would have occurred, nor does the failure to account for the sodium tetraborate in the ice bed represent an unanalyzed condition or a condition outside the plant's design basis.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

| FACILITY NAME (1) | DOCKET (2) NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
|-------------------------------------|--------------------------|----------------|----------------------|--------------------|----------|
| Donald C. Cook Nuclear Plant Unit 1 | 05000-315 | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | 2 of 2 |
| | | 1998 | 020 | 02 | |

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Conditions Prior To Event

Both units were in MODE 5.

Description Of The Event

On April 8, 1998, with Units 1 and 2 in Cold Shutdown, as a result of questions raised by personnel performing a safety system functional inspection of the Containment Spray system, it was initially believed that the Westinghouse BORDER code used to determine post-accident pH in the containment sump had not included the sodium tetraborate contained within the ice condenser ice bed. The failure to include the sodium tetraborate in the calculation could have potentially resulted in the containment sump pH value exceeding the maximum pH limit following a Loss of Coolant Accident. An ENS notification was conservatively made at 1440 hours EDT in accordance with 10CFR50.72(b)(2)(i), for an unanalyzed condition discovered while the plant was shutdown. Interim LERs 315/98-020-00, and -01 were submitted in accordance with 10CFR50.72(a)(2)(i).

In October 2000, CNP completed its investigation of the event. This investigation determined that although the BORDER Code did not account for the sodium tetraborate present in the ice, this error did not result in a condition contrary to the plant's technical specifications (T/S). In all cases the containment sump pH would have remained within the limits specified in the T/S. The calculations conservatively showed that the sump pH value would not have exceeded the post LOCA pH limits assumed in the accident analysis.

The LER 315/98-020 series is hereby retracted as no violation of T/S occurred or would have occurred, nor does the failure to account for the sodium tetraborate in the ice bed represent an unanalyzed condition or a condition outside the plant's design basis. This supplement replaces all previous revisions in their entirety.

Basis for Retraction

Subsequent investigation has determined that the T/S maximum allowed pH limit would not have been exceeded. Although the BORDER code did not include the sodium tetraborate contribution to the containment sump pH from ice melt, the identified condition had no safety significance. The plant configuration and processes regarding chemistry control are established by controlling appropriate plant chemistry that is not dependent on the BORDER code. The chemistry of the primary, secondary, refueling water storage tank, and containment spray additive tanks are controlled by additional technical specifications. The additional technical specification chemistry requirements ensure the appropriate pH levels are met following a design basis accident.