November 14, 19

Docket No. 50-333

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Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 73341)

The Commission has issued the enclosed Amendment No. 143 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 31, 1989.

The amendment updates Table 4.7-2, "Exceptions to Type C Tests," to accurately reflect the as-built configuration of the plant, to correct several editorial errors, and improve the format of the table.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

David E. LaBarge, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 143 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 14, 1989

Docket No. 50-333

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Executive Vice President - Nuclear Generation
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White Plains, New York 10601

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David E. LaBarge, Project Manager
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Division of Reactor Projects - I/II
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cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Power Authority of the State of New York (the licensee) dated May 31, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public and;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 143 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 14, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 143

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages	Insert Pages
211	211
* 211a	212
212	213
213	213a
	213b

* discard page 211a.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

The following penetrations are excepted from Type C testing requirements:

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
7A 7B 7C 7D	Main Steam	29AOV-80A 29AOV-80B 29AOV-80C 29AOV-80D 29AOV-86A 29AOV-86B 29AOV-86C 29AOV-86D	The inboard valves will be tested in the reverse direction. Pressure will be applied between the isolation valves and leakage measured. A water seal of 25 psig will be used on the inboard valve to determine the outboard valve's leak rate. (limit 11.5 SCFH at 25 psig.)
25	Drywell Purge Inlet (Air and/or Nitrogen)	27AOV-112 27AOV-131A 27AOV-131B	These valves will be tested in the reverse direction.
26A & 26B	Drywell Purge Inlet (Air and/or Nitrogen)	27AOV-113 27MOV-122	These valves will be tested in the reverse direction.
30A	Instrumentation	Various	Will not be tested as lines are sealed by process fluid.
35A	Traversing In-Core Probe	07NM-104A	This valve is an explosive shear valve which cannot be Type C tested.
35B	Traversing In-Core Probe	07NM-104B	This valve is an explosive shear valve which cannot be Type C tested.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
35C	Traversing In-Core Probe	07NM-104C	This valve is an explosive shear valve which cannot be Type C tested.
35D	Traversing In-Core Probe	07NM-104D	This valve is an explosive shear valve which cannot be Type C tested.
37A 37B 37C 37D	Control Rod Drive (Inlet)	SOV-120 SOV-123 AOV-126 CRD-138	Will not be tested as lines are sealed by process fluid.
38A 38B 38C 38D	Control Rod Drive (Outlet)	SOV-121 SOV-122 AOV-127	Will not be tested as lines are sealed by process fluid.
39A	RHR Cont. Spray	10MOV-31A	This valve will be tested in the reverse direction.
39B	RHR Cont. Spray	10MOV-31B	This valve will be tested in the reverse direction.
45	Drywell Pressure Sensing	16-1AOV-101A	This valve will be tested in the reverse direction.
50C	Instrumentation - Sensing DW Pressure	Various	These instrument root valves are tested during a Type A test.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
202B	Vacuum Breaker - Reactor Building to Suppression Chamber	27AOV-101A 27AOV-101B	These valves will be tested in the reverse direction.
205	Pressure Suppression Chamber Purge Exhaust (Air or Nitrogen)	27AOV-117 27MOV-117	These valves will be tested in the reverse direction.
210A	RHR to Suppression Pool, RCIC, Core Spray Test to Suppression Pool	10MOV-16A 10MOV-21A 10MOV-34A 10MOV-167A 13MOV-27 14MOV-5A 14MOV-26A	Will not be tested as lines are water sealed by suppression chamber water. Valve 10MOV-34A is tested during the Type C test of Penetration X-211A.
210B	RHR to Suppression Pool, HPCI, Core Spray Test to Suppression Pool	10MOV-16B 10MOV-21B 10MOV-34B 10MOV-167B 14MOV-5B 14MOV-26B 23MOV-25	Will not be tested as lines are water sealed by suppression chamber water. Valve 10MOV-34B is tested during the Type C test of Penetration X-211B.
211A	RHR to Suppression Spray Header	10MOV-38A	This valve will be tested in the reverse direction.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
211B	RHR to Suppression Spray Header	10MOV-38B	This valve will be tested in the reverse direction.
218	Torus Pressure Sensing	16-1AOV-102B	This valve will be tested in the reverse direction.
220	Torus Purge Inlet (Air and/or Nitrogen)	27AOV-116 27AOV-132A 27AOV-132B	These valves will be tested in the reverse direction.
221	RCIC - Vacuum to Torus	13RCIC-07	Will not be tested as line is sealed by suppression chamber water.
222	HPCI - Turbine Drain Trap to Torus	23HPI-13	Will not be tested as line is water sealed by suppression chamber water.
224	RCIC - Pump Suction (Torus)	13MOV-39 13MOV-41	Will not be tested as lines are water sealed by suppression chamber water.
225A	RHR - Pump Suction, RHR to Radwaste	10MOV-13A 10MOV-13C 10MOV-57 10MOV-67	Will not be tested as lines are water sealed by suppression chamber water.
225B	RHR - Pump Suction	10MOV-13B 10MOV-13D	Will not be tested as lines are water sealed by suppression chamber water.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
226	HPCI - Pump Suction (Torus)	23MOV-57 23MOV-58	Will not be tested as lines are water sealed by suppression chamber water.
227A	Core Spray - Pump Suction (Torus)	14MOV-7A	Will not be tested as line is water sealed by suppression chamber water.
227B	Core Spray - Pump Suction (Torus)	14MOV-7B	Will not be tested as line is water sealed by suppression chamber water.
228	Condensate to Torus	33CND-102	Will not be tested as line is water sealed by suppression chamber water.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

INTRODUCTION

By letter dated May 31, 1989, the Power Authority of the State of New York (PASNY or the licensee), requested changes to Facility Operating License No. DRP-59 for the James A. FitzPatrick Nuclear Power Plant. The changes would update Technical Specifications (TS) Table 4.7-2, "Exceptions to Type C Tests," to accurately reflect the as-built configuration of the plant and the tests required by Appendix J to 10 CFR Part 50. The amendment would also correct several editorial errors existing in the table and improve the general format of the table.

The letter also requested that the number of the table be changed from 3.7-2 to 4.7-2. However, this change was made in Amendment No. 134 which was issued by the NRC on July 19, 1989.

DISCUSSION

Type C Local Leak Rate Tests (LLRTs) are tests which are designed to measure the leakage rate from the primary containment isolation valves. The requirements and acceptance criteria for the tests are given in Appendix J to 10 CFR Part 50 and are implemented at the FitzPatrick Nuclear Power Plant using various procedures. Since the specific testing requirements cannot be applied to all valves which are a part of the primary containment boundary because of valve or system design, exceptions which incorporate other approved methods are necessary. These exceptions are listed in TS Table 4.7-2, along with a brief explanation of the local leak rate test which is performed.

Many of the proposed changes to the table involve replacement of a long description of the exception with a short, concise explanation. Thus, the statement: "This valve is a butterfly valve-pressurization in reverse direction and measurement of leakage will be equivalent to results from pressure applied in the same direction as that when the valve would be required to perform it safety function" would be replaced with: "These valves will be tested in the reverse direction."

The proposed TS amendment would also remove some valves from the table, thereby removing their exception status. Sample system valves and valves 13-MOV-15 and 23-MOV-15 would be removed from the table since they have been redesigned to incorporate the capability to perform the leak rate tests. Valve 13-MOV-130 (Penetration 212) would be removed from the table since it is not a primary containment boundary valve and not subject to LLRTs.

A number of valves would be added to the exceptions table. Control Rod Drive valves associated with the hydraulic control units would be added because they are sealed with process water. The traversing incore probe valves would be added because they are explosive-type shear valves and cannot be shut. Various instrumentation valves would be added because they are either sealed by process water or because they are tested during primary containment Class A leak rate tests, as applicable, depending on their location. Valves 10MOV-34A and 10MOV-34B would be added, along with a statement indicating that they are tested during the Type C LLRT of Penetration X-211A and Penetration X-211B, respectively (valves 10MOV-34A and 10MOV-38A would, therefore, be tested together and valves 10MOV-34B and 10MOV-38B would be tested together since each pair is in series and the test pressure is applied between them).

Other miscellaneous changes would add tag numbers to the valves to better identify them, replace system name "ILRT" with "Torus Pressure Sensing," replace valve number "VSM-100T" with "16-1AOV-102A" or "16-1AOV-102B," as appropriate, to incorporate modifications which were approved in Amendment No. 36, and replace valve number 27-MOV-113 with 27MOV-122 to correct the valve number which incorporates a change which occurred due to system redesign.

SUMMARY

The proposed changes to Table 4.7-2 clarify the intent of the exceptions to Type C LLRT criteria, improve the consistency of the table, correct typographical errors and upgrade the contents of the table to reflect the as-built configuration of the plant and existing procedures. The changes do not involve modification of equipment, systems, or components, nor do they relax any administrative controls or limitations imposed on existing plant equipment. The changes do not alter the conclusions of the plant's accident analysis as documented in the Final Safety Analysis Report or the NRC staff's Safety Evaluation Report. The changes are also consistent with the requirements of Appendix J to 10 CFR Part 50. Therefore, they are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR

Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 14, 1989

PRINCIPAL CONTRIBUTOR:

D. LaBarge