

March 20, 2001

MEMORANDUM TO: Stuart A. Richards, Director
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: David H. Jaffe, Senior Project Manager, Section 1 */RA/*
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF MEETING HELD ON FEBRUARY 28, 2001, WITH
REPRESENTATIVES OF TXU ELECTRIC AND WESTINGHOUSE
ELECTRIC COMPANY CONCERNING LASER WELDED SLEEVING OF
STEAM GENERATOR TUBES

On February 28, 2001, Mr. D. Jaffe, and Mr. W. Lyon, of the Nuclear Regulatory Commission (NRC) staff, met with Mr. B. Mays and Mr. S. Swilley of TXU Electric (TXU or the licensee) and Mr. W. Cullen of Westinghouse Electric Company (Westinghouse). The purpose of the meeting was to discuss the NRC staff's review of the Westinghouse reports that had been submitted in support of the laser welded sleeving of the Comanche Peak Steam Electric Station (CPSES), Unit 1, steam generators (SGs).

Following review of the February 20, 2001, NRC staff Safety Evaluation concerning CPSES, Unit 1, laser welded sleeving, TXU pointed out that the characterization of the "plus point probe" was incorrect; the "plus point probe" is an eddy current test device rather than an ultrasonic test device. The NRC staff indicated agreement with the licensee's observation. Mr. Cullen described Westinghouse's experience with SG laser welded sleeving. Mr. Cullen indicated that loss of heat transfer due to laser welded sleeving was a minor affect. He also said that the hydraulic equivalency of laser welded sleeves relative to plugs, as calculated by the SLEEVE Code, was shown by testing conservatively. There was some discussion of the test data. The meeting participants also discussed potential use of plant data, such as those resulting from Maine Yankee Atomic Power Company's laser welded sleeving of all SG tubes, to help assess calculation accuracy. The technical discussion concluded following a brief interchange regarding whether accidents and transients might be affected by laser welded sleeves.

The NRC staff indicated that additional information would be needed for the NRC staff to complete its evaluation of the Westinghouse laser welded sleeving reports. The following was deemed by the NRC staff to be useful to complete its review:

1. A summary of the laser welded sleeve test report which might be provided in proprietary and nonproprietary forms.
2. An opportunity for the NRC staff to review calculation TH-97-08, "Sleeve Code Multiplier for Excess Conservatism." Westinghouse will consider making this calculation available for NRC staff review in its Rockville, Maryland office.

3. Documentation concerning the SLEEVE Code.
4. Laser welded sleeve/flow data from other plants.
5. Insights on the affects of laser welded sleeving on accidents and transients. If determined to be needed, this information will be requested via specific questions from the NRC staff.

Following the discussion of the NRC staff's information needs, the meeting was adjourned.

Docket Nos.: 50-445 and 50-446

Enclosures: As stated

cc w/encls: See next page

3. Documentation concerning the SLEEVE Code.
4. Laser welded sleeve/flow data from other plants.
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Comanche Peak Steam Electric Station

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