November 10, 1988

Docket No. 50-333

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Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
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White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your applications transmitted by letters dated May 27 and August 10, 1988 (TAC Nos. 68354 and 69101).

The amendment revises TS Table 3.7-1 to reflect certain modifications involving containment isolation valves performed during the Reload 8/Cycle 9 refueling outage.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

David E. LaBarge, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:
1. Amendment No.118 to DPR-59
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1	PDI-1	PDI-1	OGC
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James A. FitzPatrick Nuclear
Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Power Authority of the State of New York (the licensee) dated May 27 and August 10, 1988, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.118 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 10, 1988

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages	Insert Pages
198	198
199	199
200	200
202	202
205	205

Table 3.7-1 (Cont'd)

PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT

(Numbers in parentheses are keyed to numbers on following pages: signal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
Main Steam Line	X-7A,B,C,D	AO Globe	Air and AC, DC	A	Inside	Air and spring	B,C,D,P,E	Note (1)	Open	
Main Steam Line	X-7A,B,C,D	AO Globe	Air and AC, DC	A	Outside	Air and spring	B,C,D,P,E	Note (1)	Open	
Main Steam Line Drain	X-8	MO Gate	AC	A	Inside	Ac	B,C,D,P,E	15 sec	Closed	
Main Steam Line Drain	X-8	MO Gate	DC	A	Outside	DC	B,C,D,P,E	15 sec	Closed	
From Reactor Feedwater	X-9A, B	Check	-	A	Outside	Process	Rev. flow	NA	Open	
From Reactor Feedwater	X-9A, B	Check	-	A	Inside	Process	Rev. flow	NA	Open	
Reactor Water Sample	X-41	SO Valves	AC	A	Inside	Spring	B,C	NA	Open	
Reactor Water Sample	X-41	SO Valves	AC	A	Outside	Spring	B,C	NA	Open	
Control Rod Hydraulic Return	X-36	Check	-	A	Inside	Process	Rev. flow	NA)		
Control Rod Hydraulic Return	X-36	Check	-	A	Inside	Process	Rev. flow	NA)	Opens on Rod movement and closed at all other times, Note (4)	
Control Rod Drive Exhuast	X-38	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Exhuast	X-38	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Inlet	X-37	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Inlet	X-37	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		

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Table 3.7-1 (Cont'd)
PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT
(Numbers in parentheses are keyed to numbers on following pages: signal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
Mini-purge to recirc pump	X-31Ac X-31Bc	SO Valve	Ac	C	Outside	Spring	B,F,RM	Not applicable	Open	
Mini-purge to recirc pump	X-31Ac X-31Bc	Check	Process	C	Inside	Process	Rev. flow	Not applicable	Open	
RHR Reactor Shut-down Cooling supply	X-12	MO Gate	Dc	A	Outside	Dc	A,U,F,RM	38 Sec	Closed	
RHR Reactor Shut-down Cooling supply	X-12	MO Gate	Ac	A	Inside	Ac	A,U,F,RM	38 Sec	Closed	
RHR to Suppression Spray Header	X-211A,B	MO Globe	Ac	B	Outside	Ac	G,S,RM	10 Sec	Closed	Throttling Type Valve Note (2)
RHR - Containment Spray	X-39A	MO Gate	Ac	B	Outside	Ac	G,S,RM	10 Sec	Closed	Note (2)
RHR - Containment Spray	X-39A	MO Globe	Ac	B	Outside	Ac	G,S,RM	10 Sec	Closed	Throttling Type Valve Note (2)
RHR - Containment Spray	X-39B	MO Gate	Ac	B	Outside	Ac	G,S,RM	10 Sec	Closed	Note (2)
RHR - Containment Spray	X-39B	MO Globe	Ac	B	Outside	Ac	G,S,RM	10 Sec	Closed	Throttling Type Valve Note (2)
RHR - Reactor Head Spray	X-17	MO Gate	Ac	A	Inside	Ac	A,U,F,RM	20 Sec	Closed	
RHR - Reactor Head Spray	X-17	MO Gate	Dc	A	Outside	Dc	A,U,F,RM	20 Sec	Closed	
RHR to Suppression Pool	X-210A,B	MO Globe	Ac	B	Outside	Ac	G,RM	70 Sec	Closed	Throttling Type Valve Note (2)
RHR - LPCI to Reactor	X-13A,B	MO Gate	Ac	A	Outside	Ac	RM	120 Sec	Closed	Note (10)
RHR - LPCI to Reactor	X-13A,B	MO Globe	Ac	A	Outside	Ac	RM	90 Sec	Open	Throttling Type Valve Note (10)
RHR - LPCI to Reactor	X-13A,B	AO Check	--	A	Inside	Process	Rev. flow	Not applicable	Closed	Testable check Valve (3,16)
RHR pump suction from suppression pool	X-225A,B	MO Gate	Ac	B	Outside	Ac	RM	Not applicable	Open	

Table 3.7-1 (Cont'd)

PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT

(Numbers in parentheses are keyed to numbers on following pages: signal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
Standby Liquid Control	X-42	Check	--	A	Outside	Process	Rev. flow	Not applicable	Closed	
Standby Liquid Control	X-42	Check	--	A	Inside	Process	Rev. flow	Not applicable	Closed	
Reactor Water Clean-up from Reactor	X-14	MO Gate	Ac	A	Inside	Ac	A,J,RM,F	20 Sec	Open	
Reactor Water Clean-up from Reactor	X-14	MO Gate	Dc	A	Outside	Dc	A,V,Y,J,RM,F	20 Sec	Open	
Reactor Water Clean-up (Warm-up)	X-14	MO Gate	Dc	A	Outside	Dc	A,V,Y,J,RM,F	10 Sec	Closed	
Reactor Water Clean-up Return	X-9A	MO Gate	Ac	A	Outside	Ac	A,F,J,RM	20 Sec	Open	
RCIC - Turbine Steam Supply	X-10	MO Gate	Ac	A	Inside	Ac	K,RM	15 Sec	Open	Opens on Sig B, Line Break Sig K overrides to close valves
RCIC - Turbine Steam Supply	X-10	MO Gate	Dc	A	Outside	Dc	K,RM	15 Sec	Open	
RCIC - Turbine Exhaust	X-212	Check	Fwd flow	B	Outside	Process	Rev. flow	--	Closed	
RCIC - Minimum Pump Flow	X-210A	MO Globe	Dc	B	Outside	Dc	K,RM	5 Sec	Closed	
RCIC - Pump Discharge	X-9A	MO Gate	Dc	B	Outside	Dc	RM	Not applicable	Closed	
RHR to Radwaste	X-225A	MO Gate	Ac	B	Outside	Ac	A,F,RM	24 Sec	Closed	
RHR to Radwaste	X-225A	MO Gate	Dc	B	Outside	Dc	A,F,RM	24 Sec	Closed	
RCIC - Vacuum Pump Discharge	X-226	Check	Fwd flow	B	Outside	Process	Rev. flow	--	Closed	
RCIC - Pump Suction	X-224	MO Gate	Dc	B	Outside	Dc	RM	Not applicable	Closed	
RCIC - Pump Suction	X-224	MO Gate	Dc	B	Outside	Dc	RM	Not applicable	Closed	

Table 3.7-1 (Cont'd)
PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT
(Numbers in parentheses are keyed to numbers on following pages: signal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
HPCI - Turbine Exhaust	X-214	Check	Fwd flow	B	Outside	Process	Rev. flow	Not applicable	Open	Closes on Rev. flow or low exhaust pressu
HPCI - Turbine Exhaust	X-214	Check	Fwd flow	B	Outside	Process	Rev. flow	Not applicable	Open	
HPCI - Pump Suction	X-226	MO Gate	Dc	B	Outside	Dc	L,RM	Not applicable	Closed	
HPCI - Pump Suction	X-226	MO Gate	Dc	B	Outside	Dc	L,RM	Not applicable	Closed	
HPCI - Pump Discharge	X-9B	MO Gate	Dc	B	Outside	Dc	RM	Not applicable	Closed	
HPCI - Turbine Exhaust Drain	X-222	Stop Check	Fwd flow	B	Outside	Process	Rev. flow	Not applicable	Closed	
HPCI - Minimum Pump Flow	X-210B	Check	Fwd flow	B	Outside	Process	Rev. flow	Not applicable	Closed	
HPCI - Minimum Pump Flow	X-210B	MO Globe	Dc	B	Outside	Dc	L,RM	10 Sec	Closed	
DRYWELL ATMOSPHERIC CONTROL AND SERVICES										
Instrument Air to Drywell	X-22	Check	Process	C	Inside	Process	Rev. flow	Not applicable	Open	
Instrument Air to Drywell	X-22	SO Valve	Spring	C	Outside	Ac	RM	Not applicable	Open	Fail in d position to ensure adequate pneumatic supply
Drywell Purge Inlet	X-25, X-71	AO Butterfly	Air/Ac	B	Outside	Spring	F,A,Z,RM	5 sec	Closed	
Drywell Purge Inlet	X-25, X-71	AO Butterfly	Air/Ac	B	Outside	Spring	F,A,Z,RM	5 sec	Closed	
Drywell Main Exhaust	X-26A,B	AO Butterfly	Air/Ac	B	Outside	Spring	F,A,Z,RM	5 sec	Closed	

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Table 3.7-1 (Cont'd)
PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT
(Numbers in parentheses are keyed to numbers on following pages: signal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
CAD Supply to Instrument Air to Drywell	X-57c	SO Valve	Spring	C	Outside	Ac	RM	Not applicable	Open	) Fail in open) position ( ) ensure adequate) pneumatic supply
CAD Supply to Instrument Air to Drywell	X-57c	Check	Process	C	Inside	Process	Rev. flow	Not applicable	Closed	
Drywell Atmosphere Sample Supply	X-31Ad	SO Valve	Ac	B	Outside	Spring	F,A,Z,RM	Not applicable	Open	
Drywell Atmosphere Sample Supply	X-31Bd	SO Valve	Ac	B	Outside	Spring	F,A,Z,RM	Not applicable	Open	
Drywell Atmosphere Sample Return	X-52a	SO Valve	Ac	B	Outside	Spring	F,A,Z,RM	Not applicable	Open	
Drywell Atmosphere Sample Return	X-55b	SO Valve	Ac	B	Outside	Spring	F,A,Z,RM	Not applicable	Open	

Amendment No. ~~90~~, ~~98~~, ~~108~~, 118

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

INTRODUCTION

By letters dated May 27 and August 10, 1988, the Power Authority of the State of New York (the licensee) requested changes to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The change would revise Table 3.7-1, "Process Pipeline Penetrating Primary Containment," of the Technical Specifications (TS) to reflect six plant modifications which will be completed during the 1988 refueling outage currently underway.

DESCRIPTION

The modifications involve the Primary Containment Radioactivity Monitoring System, service and breathing air supplies to the drywell, Reactor Water Sample containment isolation valves, Residual Heat Removal (RHR) system containment isolation valves, and Reactor Water Cleanup (RWCU) System containment isolation valves. Specifically the licensee requested the following changes to the TS:

1. Add two containment penetrations (X-31Ad and X-55b) and four containment isolation valves for the redundant primary containment radioactivity monitor.

2. Eliminate service and breathing air supplies to drywell.

3. Replace RHR motor-operated gate valves (10MOV-31A and 10MOV-31B) with motor-operated globe valves.

4. Replace Reactor Water Sample air-operated globe valves (02-2AOV-39 and 02-2AOV-40) with solenoid-operated valves.

5. Add containment isolation signal F (high drywell pressure) to three existing RWCU system containment isolation valves (12MOV-15, 12MOV-18 and 12MOV-80).

6. Add an isolation valve (12MOV-069) outside containment in the RWCU system.

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EVALUATION

1. Primary Containment Radioactivity Monitoring System (PCRMS) (Gaseous and Particulate)

The FitzPatrick plant has redundant primary containment radioactivity monitors. PCRMS instruments monitor both gaseous and particulate radioactivity levels during normal plant operation. A single pair of supply and return lines services both monitors (i.e., both monitors draw primary containment [drywell] atmosphere through a single supply line and exhaust through a single return line). The licensee proposes to install a second pair or PCRMS sample lines to improve system reliability. Each PCRMS monitor will have its own supply and return lines. This modification will eliminate the possibility that both monitors could be inoperable as a result of a single line failure. When the modification is complete, the following arrangement will exist:

Penetration	Function	Valves
X-31Ad	Existing Supply	27SOV-135A and 27SOV-135C
X-55b	Existing Return	27SOV-125B and 27SOV-125D
X-31Bd	New Supply	27SOV-135D and 27SOV-135B
X-52a	New Return	27SOV-125C and 27SOV-125A

Further, the licensee is required to revise the FSAR to reflect the new valve arrangement. The staff finds the new arrangement acceptable.

2. Service and Breathing Air Supplies to Drywell

The licensee proposes to cut and cap pipe lines associated with containment penetration X-21 and X-61 for breathing air and service air, respectively. These penetrations were designed and installed to supply breathing and service air for personnel and equipment in the drywell during maintenance. Currently, the licensee is using a portable air supply. The licensee stated that the cutting and capping of these air lines will eliminate a potential source of air leaks into FitzPatrick's inerted drywell. The work will be performed in accordance with Section III of the ASME Code. On the basis of its review the staff concludes that this modification will reduce the potential for containment leakage following a postulated accident. Therefore, the staff finds this modification acceptable.

3. RHR System Containment Isolation Valve Change

The licensee proposes to replace two RHR system containment spray motor-operated gate valves (10MOV-31A and 10MOV-31B) with motor-operated globe valves. This change will improve the operators' ability to throttle containment spray flow during certain accident conditions. The flow characteristics of globe valves make them suitable for flow throttling. Even though maximum containment spray flow will be reduced from approximately 10950 gpm to 6700 gpm, the licensee has determined that this reduction in maximum flow represents no significant reduction in the margin of safety because the containment spray system will still be able to perform its intended function. Containment spray has three primary functions: (1) Assure primary containment integrity, (2) Assure that drywell/torus temperatures and

pressures remain within equipment qualification limits, and (3) Reduce or eliminate chugging. The closure time of the new globe valves is the same as that of existing valves, therefore flow dynamics are not changed. The isolation signals used to actuate these containment isolation valves remain unchanged. Environmental qualification analyses are unaffected. Therefore, the staff concludes that containment spray flow resulting from replacing gate valves with globe valves is acceptable.

4. Reactor Water Sample Containment Isolation Valves Change

The licensee proposes to replace two Reactor Water Sample containment isolation air-operated globe valves (02-2AOV-39 and 02-2AOV-40) with solenoid-operated valves (02-2SOV-39 and 02-2SOV-40). The existing globe valves open by means of pneumatic air pressure and electrical AC power. The new solenoid valves will use AC power only and will provide positive valve position indication. The globe valves are being replaced to comply with both the licensee's commitment regarding post-accident instrumentation and with Regulatory Guide 1.97, Rev. 2. Regulatory Guide 1.97 states the purpose of primary containment isolation valve position indication as "accomplishment of isolation." Primary containment isolation valve position provides information to indicate that plant safety functions are being achieved. The modification satisfies this criterion. Reactor Water Sample containment isolation valves are used to draw a small sample of reactor water for analysis. The valves are normally open to provide flow to the crack arrest verification subsystem of the hydrogen water chemistry system and fail to the close position. The isolation signals used to actuate these valves are not changed by this modification. Also, the 10 CFR Part 50, Appendix J, criteria for testing of new valves will not change. Therefore, the staff concludes that this modification is acceptable.

5. Reactor Water Cleanup Containment Isolation Valves

The licensee proposed two modifications associated with the Reactor Water Cleanup (RWCU) System containment isolation valves in response to NUREG-0737, Item II.E.4.2 "Containment Isolation Dependability." The first of the two modifications involves adding an isolation signal F (high drywell pressure) to the three existing containment isolation valves (12MOV-15, 12MOV-18, 12MOV-80). These three valves currently actuate on Primary Containment and Reactor Vessel Isolation Control System (PCRVICS) signals that indicate reactor vessel low water level and RWCU line break. The containment isolation signal A (reactor vessel low water level) is indicative of either a loss of feedwater or a loss of coolant. Isolation signal F (high drywell pressure) indicates a breach in the RCS inside the drywell. Isolation signal J (RWCU system equipment room high temperature) detects a RWCU system line break. The two isolation signals A and J are not diverse because high RCWCU space temperature will not detect a reactor coolant system break within the drywell. When this modification (addition of isolation signal F to RWCU containment isolation valve) is complete, these valves will be actuated using diverse isolation signal as recommended in Section 6.2.4 of the Standard Review Plan (NUREG-0800). The staff considers this modification to be acceptable.

The second modification involves the addition of an isolation valve (12MOV-069) outside containment in the RWCU system. This motor-operated valve will be installed in the RWCU return line to the feedwater system and will replace the TS isolation function of the check valve. The valve will be located in the Reactor Building between the Reactor Coolant Isolation Cooling (RCIC) discharge thermal sleeve and an existing manual valve (12RWC-63). The valve position indication will be provided in the main control room and on the emergency plant and information computer (EPIC) system. A remote-manual switch will be installed in the main control room. The valve will fail in the "as-is" position upon loss of actuation power. Electric power will be supplied from a safety-related motor control center. This new valve will actuate on diverse isolation signals of either reactor vessel low water level or high drywell pressure as discussed above. The valve and its associated control circuitry are classified as safety-related, Seismic Class I, QA Category I and electrical Class 1E and is subject to 10 CFR Part 50, Appendix J, testing criteria. Currently, the RWCU return line is isolated by the single check valve (12RWC-62). The addition of the new motor-operated containment isolation valve on the RWCU return line in addition to a check valve will improve the leak tight integrity of the system in the event of an accident. A motor-operated valve also provides valve position indication not available with a check valve. Thus this modification of the RWCU system will comply with General Design Criterion 55 of Appendix A to 10 CFR Part 50 - Reactor Coolant Pressure Boundary Penetrating Containment. Therefore, the staff considers this modification acceptable.

In addition to the modifications listed above, the licensee revised TS Table 3.7-1 to update and correct RWCU entries. The maximum allowable closure time for containment isolation valves (12MOV-15 and 12MOV-18) associated with the RWCU system supply lines changed from 30 seconds to 20 seconds. The licensee stated that this change was required to reflect assumptions made in the FitzPatrick environmental qualification analyses. Faster valve closing time reduces the quantity of reactor coolant lost through the break. Consequently, the maximum temperature, humidity and radiation levels around the break are also reduced. The staff considers this change acceptable. Also, the licensee added an isolation signal Y (Standby Liquid Control System Actuated) for valve 12MOV-18 which was inadvertently omitted from TS Table 3.7-1. The signal Y was installed on RWCU valve (12MOV-18) as part of the original FitzPatrick design. The staff considers this feature acceptable.

On the basis of the licensee's proposed modifications, the staff concludes that the proposed changes to Table 3.7-1 of the plant Technical Specifications are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in

individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 10, 1988

PRINCIPAL CONTRIBUTOR:

R. Anand