

UNITED STATES **NUCLEAR REGULATORY COMMISSION**

WASHINGTON, D.C. 20555-0001

March 16, 2001

Mr. Mark Reddemann Site Vice President Kewaunee and Point Beach Nuclear Power Plants Nuclear Management Company, LLC 6610 Nuclear Road Two Rivers, WI 54241

SUBJECT: KEWAUNEE NUCLEAR POWER PLANT - REQUEST FOR ADDITIONAL

INFORMATION RELATED TO RELOAD SAFETY EVALUATION METHODS

TOPICAL REPORT, WPSRSEM-NP, REVISION 3 (TAC NO. MB0306)

Dear Mr. Reddemann:

By letter dated October 12, 2000, Nuclear Management Company, LLC (NMC or the licensee) forwarded a topical report WPSRSEM-NP, Revision 3, entitled, "Reload Safety Evaluation Methods for Application to Kewaunee" and requested Nuclear Regulatory Commission (NRC) staff review and approval. Also, the topical report is intended to be applicable to Kewaunee reload cycles after and including Cycle 25, presently scheduled to commence in the fall of 2001.

The topical report reflects the following:

- Editorial changes, including corrections to the limiting directions of core physics parameters and clarification of the definition of core physics parameters.
- Changes made to incorporate the CONTEMPT code for containment analysis. CONTEMPT is currently described for this purpose in the Kewaunee updated safety analysis report (USAR).
- The adoption of the GOTHIC code for containment analysis.
- Changes in Reload Safety Evaluation Methods due to Large Break Loss-of-Coolant Accident Upper Plenum Injection Analysis.
- The adoption of RETRAN- 3D for use in the 2D mode for system analysis.
- The extension of the VIPRE-01 code to reflect changes in fuel design.

The NRC staff finds that the additional information identified in Enclosure 1 is needed.

A draft of the request for additional information (RAI) (Enclosure 2) was e-mailed to Mr. G. Riste (NMC) and discussed by telephone call on March 12, 2001. Another phone call was held on March 14, 2001, between the NRC staff and the NMC staff to discuss the questions to ensure that there was no misunderstanding.

Also, the phone call on March 14, 2001, established that 21 days after the date of this letter, would be a mutually agreeable response date.

Please contact me at (301) 415-1446 if future circumstances should require a change in this response date.

Sincerely,

/RA/

John G. Lamb, Project Manager, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-305

Enclosures: 1. Request for Additional Information

2. E-mail with Draft Request for Additional Information

cc w/encls: See next page

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Please contact me at (301) 415-1446 if future circumstances should require a change in this response date.

Sincerely,

John G. Lamb, Project Manager, Section 1

Project Directorate III

Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-305

Enclosures: 1. Request for Additional Information

2. E-mail with Draft Request for Additional Information

cc w/encls: See next page

Kewaunee Nuclear Power Plant

CC:

Foley & Lardner
ATTN: Bradley D. Jackson
One South Pinckney Street
P.O. Box 1497
Madison, WI 53701-1497

Chairman Town of Carlton Route 1 Kewaunee, WI 54216

Gerald Novickus, Chairman Kewaunee County Board Kewaunee County Courthouse Kewaunee, WI 54216

Attorney General 114 East, State Capitol Madison, WI 53702

U.S. Nuclear Regulatory Commission Resident Inspectors Office Route #1, Box 999 Kewaunee, WI 54216

Regional Administrator - Region III U.S. Nuclear Regulatory Commission 801 Warrenville Road Lisle, IL 60532-4531

James D. Loock, Chief Engineer Public Service Commission of Wisconsin 610 N. Whitney Way Madison, WI 53707-7854

Michael D. Wadley Chief Nuclear Officer Nuclear Management Company, LLC 700 First Street Hudson, WI 54016 Nuclear Asset Manager Wisconsin Public Service Corporation 600 N. Adams Street Green Bay, WI 54307-9002

Plant Manager Kewaunee Nuclear Power Plant Nuclear Management Company, LLC North 490, Highway 42 Kewaunee, WI 54216-9511

REQUEST FOR ADDITIONAL INFORMATION REGARDING

KEWAUNEE RELOAD SAFETY EVALUATION METHODS TOPICAL REPORT,

WPSRSEM-NP, REVISION 3 (TAC NO. MB0306)

The Nuclear Regulatory Commission (NRC) staff finds that the following additional information is needed:

- 1. Please provide one copy of Reference A12, WCAP-15427, "Development and Qualification of a GOTHIC Containment Evaluation Model for the Kewaunee Nuclear Power Plant," R. Ofstun, July 2000.
- 2. To the extent that the above report does not contain this information, provide for the GOTHIC model:
 - 2.1 Details of how the Kewaunee containment will be modeled for a loss-of-coolant (LOCA) and a main steamline break.
 - 2.2 Assumptions used for licensing basis calculations including containment initial conditions of temperature, pressure and humidity; heat transfer correlations; modeling of the velocity of the containment atmosphere and its effect on heat transfer; timing of engineered safety feature (ESF) equipment; assumptions of spray behavior (droplet size distribution, spray effectiveness); interactions of containment atmosphere with the sump following a LOCA; modeling of residual heat removal (RHR) heat exchanger; modeling of fan coolers; modeling of containment structures; any other modeling assumptions which have a significant effect on the calculated pressure and temperature of the containment.
 - 2.3 Describe or provide a reference on how calculations of the mass and energy release (including entrainment) to the containment are performed.
 - 2.4 List and describe the conservatisms in the above modeling.
 - 2.5 List any other licensing basis uses of the GOTHIC code beside LOCA and main steamline break analyses.
- 3. Is the Kewaunee use of GOTHIC consistent with the guidance of generic letter (GL) 83-11, "Licensee Qualification for Performing safety Analyses in Support of Licensing Actions," [February 8, 1983] and GL 83-11 Supplement 1, "Licensee Qualification for Performing Safety Analyses" [June 24, 1999]?

REQUEST FOR ADDITIONAL INFORMATION WISCONSIN PUBLIC SERVICE CORPORATION RELOAD SAFETY EVALUATION METHODS TOPICAL REPORT WPSRSEM-NP, Revision 3

- 1. Please provide one copy of Reference A12, WCAP-15427, "Development and Qualification of a GOTHIC Containment Evaluation Model for the Kewaunee Nuclear Power Plant," R. Ofstun, July 2000
- 2. To the extent that the above report does not contain this information, provide for the GOTHIC model:
 - 2.1 Details of how the Kewaunee containment will be modeled for a LOCA and a main steam line break
 - 2.2 Assumptions used for licensing basis calculations including containment initial conditions of temperature, pressure and humidity; heat transfer correlations; modeling of the velocity of the containment atmosphere and its effect on heat transfer; timing of ESF equipment; assumptions of spray behavior (droplet size distribution, spray effectiveness); interactions of containment atmosphere with the sump following a LOCA; modeling of RHR heat exchanger; modeling of fan coolers; modeling of containment structures; any other modeling assumptions which have a significant effect on the calculated pressure and temperature of the containment.
 - 2.3 Describe or provide a reference on how calculations of the mass and energy release (including entrainment) to the containment are performed.
 - 2.4 List and describe the conservatisms in the above modeling
 - 2.5 List any other licensing basis uses of the GOTHIC code beside LOCA and main steam line break analyses.
- 3. Does the Kewaunee use of GOTHIC comply with the guidance of GL 83-11, "Licensee Qualification for Performing safety Analyses in Support of Licensing Actions," [February 8, 1983] and GL 83-11 Supplement 1, "Licensee Qualification for Performing Safety Analyses?" [June 24, 1999]

From:

John Lamb

To:

internet: griste@wpsr.com

Date:

3/12/01 4:30PM

Subject:

Fwd: Questions on 10/12/200 report concerning containment

Gerry,

Attached are draft questions concerning the GOTHIC code of your Reload Amendment submittal. Please review the questions. I will call you to set up a conference call to ensure that we understand the questions and to set up a response date, then I will send a formal RAI letter to you.

John

From: Richard Lobel
To: John Lamb

Date: 3/12/01 4:20PM

Subject: Questions on 10/12/200 report concerning containment

I have revised the questions from the ones I sent to you earlier. They are ready to transmit to the licensee.