



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 9, 2001

Mr. James R. Morris
Site General Manager
Monticello Nuclear Generating Plant
Nuclear Management Company, LLC
2807 West County Road 75
Monticello, MN 55362-9637

**SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT - ISSUANCE OF AMENDMENT
RE: REVISION OF STANDBY LIQUID CONTROL SYSTEM SURVEILLANCE
REQUIREMENTS (TAC NO. MB1021)**

Dear Mr. Morris:

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications in response to your application dated January 10, 2001.

The amendment removes the standby liquid control (SLC) pump flow surveillance requirement to recycle demineralized water to the test tank and changes the testing frequency of the SLC pump capacity test from monthly to quarterly.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Carl F. Lyon, Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures: 1. Amendment No. 118 to DPR-22
2. Safety Evaluation

cc w/encls: See next page

NRR-058

March 9 2001

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RBouling BBurgess, RGN-III *Previously concurred

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NAME	FLyon	RBouling	RCaruso	RHoefling	CCraig
DATE	3/8/01	3/8/01	2/27/01	3/5/01	3/8/01

DOCUMENT NAME: G:\PDIII-1\MONTICEL\amdb1021.wpd

OFFICIAL RECORD COPY

Monticello Nuclear Generating Plant

cc:

**J. E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington, DC 20037**

**U.S. Nuclear Regulatory Commission
Resident Inspector's Office
2807 W. County Road 75
Monticello, MN 55362**

**Site Licensing Manager
Monticello Nuclear Generating Plant
Nuclear Management Company, LLC
2807 West County Road 75
Monticello, MN 55362-9637**

**Robert Nelson, President
Minnesota Environmental Control
Citizens Association (MECCA)
1051 South McKnight Road
St. Paul, MN 55119**

**Commissioner
Minnesota Pollution Control Agency
520 Lafayette Road
St. Paul, MN 55155-4194**

**Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60532-4351**

**Commissioner
Minnesota Department of Health
717 Delaware Street, S. E.
Minneapolis, MN 55440**

**Douglas M. Gruber, Auditor/Treasurer
Wright County Government Center
10 NW Second Street
Buffalo, MN 55313**

**Commissioner
Minnesota Department of Commerce
121 Seventh Place East
Suite 200
St. Paul, MN 55101-2145**

**Adonis A. Neblett
Assistant Attorney General
Office of the Attorney General
445 Minnesota Street
Suite 900
St. Paul, MN 55101-2127**

**Michael D. Wadley
Chief Nuclear Officer
Nuclear Management Company, LLC
700 First Street
Hudson, WI 54016**

**Nuclear Asset Manager
Xcel Energy, Inc.
414 Nicollet Mall
Minneapolis, MN 55401**

October 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-22

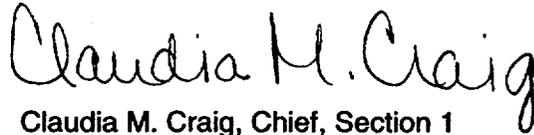
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated January 10, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 9, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE

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INSERT

93

3.0 LIMITING CONDITIONS FOR OPERATION

3.4 STANDBY LIQUID CONTROL SYSTEM

Applicability:

Applies to the operating status of the standby liquid control system.

Objective:

To assure the availability of an independent reactivity control mechanism.

Specification:

A. System Operation

1. The standby liquid control system shall be operable at all times when fuel is in the reactor and the reactor is not shut down by control rods, except as specified in 3.4.A.2.
2. From and after the date that a redundant component is made or found to be inoperable, reactor operation is permissible only during the following 7 days provided that the redundant component is operable.

3.4/4.4

4.0 SURVEILLANCE REQUIREMENTS

4.4 STANDBY LIQUID CONTROL SYSTEM

Applicability:

Applies to the periodic testing requirements for the standby liquid control system.

Objective:

To verify the operability of the standby liquid control system.

Specification:

- A. The operability of the standby liquid control system shall be verified by performance of the following tests:
1. At least once per quarter -
Pump minimum flow rate of 24 gpm shall be verified against a system head of 1275 psig when tested pursuant to Specification 4.15.B. Comparison of the measured pump flow rate against equation 2 of paragraph 3.4.B.1 shall be made to demonstrate operability of the system in accordance with the ATWS Design Basis.
 2. At least once during each operating cycle -
 - a. Manually initiate one of the two standby liquid control systems and pump demineralized water into the reactor vessel. This test checks explosion of the charge associated with the tested system, proper operation of the valves and pump capacity. Both systems shall be tested and inspected, including each explosion valve in the course of two operating cycles.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-22

NUCLEAR MANAGEMENT COMPANY, LLC

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By application dated January 10, 2001, the Nuclear Management Company, LLC (NMC, or the licensee), requested changes to the Technical Specifications (TSs) for Monticello Nuclear Generating Plant. The proposed amendment would remove the standby liquid control (SLC) pump flow surveillance requirement (SR) to recycle demineralized water to the test tank and change the testing frequency of the SLC pump capacity test from monthly to quarterly.

2.0 EVALUATION

2.1 Test Tank Recycle Requirement

The licensee proposes to revise SR 4.4.A.1 to delete the requirement to recycle demineralized water to the test tank. The licensee states that it is not necessary to recycle the test tank on a monthly basis to obtain the SLC pump flow rate or to determine pump operability. The pump flow rate is determined by the volumetric change in test tank level with a suction source from the demineralized water system. The test tank is not used as a suction source in this test. The staff previously approved this method for Monticello in a safety evaluation and letter from W. Dean (NRC) to R. Anderson (licensee), "Approval of Third Ten-Year Inservice Testing Program," dated July 6, 1993.

The test tank recycle requirement periodically demonstrates the ability of the SLC pump to successfully draw from a vented suction source like the SLC boron tank. The vented draw demonstration will continue to be performed once per operating cycle in accordance with SR 4.4.A.2 (the reactor vessel injection test). The test tank recycle requirement is not typical of boiling-water reactors with similar SLC systems, and is not a requirement of Standard Technical Specifications (STS), NUREG-1433. Since the test tank recycle requirement is not required to demonstrate operability of the SLC pump, and the vented draw demonstration is performed by SR 4.4.A.2, the staff concludes that the removal of the monthly test tank recycle requirement will not adversely affect SLC system reliability. The pump flow test is unaffected by this change, and the SLC pump's ability to draw from a vented source will continue to be demonstrated by SR 4.4.A.2 at a frequency that is consistent with SLC system design and operation and with STS. Therefore, the proposed change is acceptable.

2.2 SLC Pump Capacity Test

The licensee proposes to revise the frequency of SR 4.4.A.1 from "at least once per month" to "at least once per quarter," and to add the words, "when tested pursuant to Specification 4.15.B.," so that the SR is revised to read, "Pump minimum flow rate of 24 gpm shall be verified against a system head of 1275 psig when tested pursuant to Specification 4.15.B. . . ." The proposed change allows the licensee to use inservice testing of the SLC pumps to satisfy the requirements of the SLC pump capacity test. SR 4.15.B requires that "Inservice Testing of Quality Group A, B, and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2, and 3 pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. . . ." SR 4.15.B is applicable to the SLC pumps. The staff has previously accepted the quarterly SLC pump inservice testing as a sufficient means of demonstrating component operability, measuring performance trends, and detecting incipient failures by indicating abnormal performance, as noted in the Bases for STS SR 3.1.7.7. The quarterly test frequency is also consistent with Section XI of the American Society of Mechanical Engineers Code requirements for safety-related pumps at nuclear power plants. The current capacity test procedure uses the same steps to demonstrate pump flow rate for the monthly TS requirement of SR 4.4.A.1 as the quarterly inservice testing requirement of SR 4.15.B; therefore, there is no significant change proposed to the testing methodology. Since the quarterly inservice testing of the SLC pumps required by SR 4.15.B is sufficient to demonstrate operability, measure performance trends, and detect incipient failures, and is consistent with the STS, the proposed change is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (66 FR 9386). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Lyon

Date: March 9, 2001