

March 21, 2001

LICENSEE: Rochester Gas & Electric Corporation

FACILITY: R. E. Ginna Nuclear Power Plant

SUBJECT: SUMMARY OF MEETING ON FEBRUARY 28, 2001, WITH ROCHESTER GAS AND ELECTRIC CORPORATION REGARDING A FUTURE CONTROL ROOM MODIFICATION OF THE R. E. GINNA NUCLEAR POWER PLANT (TAC NO. MB1152)

On February 28, 2001, representatives of the Rochester Gas and Electric Corporation (RG&E or the licensee) met with members of the NRC staff in Rockville, Maryland. RG&E had requested this meeting in order to brief the NRC staff on its plans to modify the R. E. Ginna control room. This would be a voluntary initiative to improve the Ginna control room ventilation system. A list of attendees is provided as Enclosure 1 and a copy of the handouts provided by RG&E is provided as Enclosure 2.

The licensee explained the reason for the meeting as informing the staff of their planned upgrade of the Ginna control room ventilation system. The current system provides some operator inconvenience because of frequent events that require going into the recirculation mode. Also, it is difficult to schedule maintenance; an upgrade of the system would allow the system to continue operation for an additional 20 years. The licensee's current system meets the current licensing basis and the current source term curves are planned to be maintained.

The X/Q values appear low; however, the NRC staff believes the values should be researched and verified to be correctly calculated. The assumed unfiltered in-leakage is at the low end of the range of in-leakage experienced at other plants; however, the licensee believes the rate is achievable. The NRC staff would prefer that the in-leakage test be performed to verify the assumptions. The assumption of passive residual heat removal pump failure is not necessarily required and the NRC would accept a submittal that did not include a passive failure in the large-break loss-of-coolant accident case calculation scenario. However, in such case the assumed ECCS system leakage rate should be a value 2X the "trigger value" that initiates corrective action under the ECCS leakage control program.

A number of radiological analyses will be performed for the majority of Chapter 15 events. Smoke removal and control room egress per Appendix E of Nuclear Energy Institute (NEI) 99-25 is acceptable and purge mode of operation to remove smoke from the control room is acceptable; however, the location of the exhaust point of that smoke warrants review. The proposed R-22 refrigerant limits are conservative enough and acceptable. The tornado missile accident will be considered although this may be considered a non-credible event. Design parameters such as charcoal filter efficiency, system flow rate, in-leakage and cooling options will be changed. In order to provide higher carbon filter efficiencies small non-safety electric

heaters may be provided in the standby carbon filter systems to assure dry filters free from condensation, humid air, etc. The cooling system, even though not included in the Technical Specifications (TSs), will be periodically tested for cooling capacity to assure the design is retained.

Changes will be proposed for the Control Room Emergency Air Treatment System (CREATS), limiting condition for operation 3.7.9. The use of control room in-leakage testing requirements within the TSs is still being discussed between NRC, NEI and the industry. Temperature control TSs are not currently being considered.

The new control room ventilation system will include redundant control room ECCS and redundant emergency control room air treatment systems. The current CREATS system will be retained for normal operation. The emergency systems will be new and will be outside the control room envelope. The NRC staff recommended that the licensee should consider modifications of expanding the control room envelope to include these emergency systems.

Since the current system is being maintained, no exact dates are being proposed for the need for the TSs changes. The licensee intends on installing most of the new systems during the spring 2002 refueling outage. The remainder of the system can be installed with the plant operating. No specific date has been established for implementing the new system; however the licensee intends to address this in their submittal for an amendment which will be approximately December 1, 2001.

/RA/

Guy S. Vissing, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket 50-244

Enclosures: As stated

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R.E. Ginna Nuclear Power Plant

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ATTENDANCE LIST
FEBRUARY 28, 2001, MEETING WITH ROCHESTER GAS & ELECTRIC CORPORATION
CONCERNING CONTROL ROOM MODIFICATIONS

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Guy S. Vissing	NRC/NRR/DLPM/PDI-1
Peter Bamford	RG&E
Dan Crowley	RG&E
George Wrobel	RG&E
Tom Harding	RG&E
Kris Parczewski	NRC/NRR/BE\EMCB
Harold Walker	NRC/NRR/DSSA/SPLB
John Segala	NRC/NRR/DSSA/SPLB
John Hayes	NRC/NRR/DSSA/SPSB
Leta Brown	NRC/NRR/DSSA/SPSB
Jason Jang	Region I
Francis Akstulewicz	NRC/NRR/DSSA/SRXB
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