



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 26, 1995

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: ISSUANCE OF AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE
NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M91753)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated March 13, 1995.

This amendment revises Technical Specification (TS) 4.4.2.4.a to replace specific leakage rate testing frequencies for containment isolation valves with a reference to 10 CFR Part 50, Appendix J, as modified by approved exemption.

This amendment is necessary to support a one-time exemption from two parts of 10 CFR Part 50, Appendix J, Section III.D.3. First, your application requests an exemption from Appendix J and the corresponding TS in performing Type C tests during the 1995 refueling outage except for isolation valves which have maintenance performed or valves which have not demonstrated acceptable leakage during the past two leakage rate tests. Second, your application requests an exemption from Appendix J and the corresponding TS in performing Type C tests within a 2-year interval, as required by the regulation. Your application requests up to a 1-month extension of the 2-year interval for 129 containment isolation valves.

The NRC staff has issued and granted similar requests in the past. On February 2, 1994, the NRC staff granted a similar exemption to the River Bend Station licensee, and by a letter dated April 29, 1987, the NRC staff granted a similar request to the Washington Public Power Supply System, Unit 2, licensee.

The NRC staff's review finds that the revised TS to replace specific leakage rate testing frequencies for containment isolation valves is acceptable as concluded in the enclosed Safety Evaluation.

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April 26, 1995

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,



Allen R. Johnson, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 59 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

DATED: April 26, 1995

AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR POWER PLANT

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

J. Zwolinski, 14/H/3

L. Marsh

S. Little

A. Johnson

R. Lobel

OGC

D. Hagan, T-4 A43

G. Hill (2), T-5 C3

C. Grimes, 11/E/22

ACRS (4)

OPA

OC/LFDCB

PD plant-specific file

J. Linville, Region I

cc: Plant Service list

Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated March 13, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 59 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 26, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove
4.4-7

Insert
4.4-7

- b. If repairs are not completed and conformance to the acceptance criterion of 4.4.2.2 is not demonstrated within 48 hours, the reactor shall be shutdown and depressurized until repairs are effected and the local leakage meets the acceptance criterion.
- c. If it is determined that the leakage through a mini-purge supply and exhaust line is greater than 0.05 La an engineering evaluation shall be performed and plans for corrective action developed.

4.4.2.4 Test Frequency

- a. Except as specified in b. and c. below, individual penetrations and containment isolation valves shall be tested in accordance with 10 CFR 50, Appendix J, as modified by approved exemptions.
- b. The containment equipment hatch, fuel transfer tube, steam generator inspection/maintenance penetration, and shutdown purge system flanges shall be tested at each refueling shutdown or after each use, if that be sooner.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO APPENDIX J, EXEMPTION REQUEST AND
AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated March 13, 1995, the Rochester Gas and Electric Corporation (RG&E or the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant (Ginna), Technical Specifications (TSs). The requested changes would revise TS 4.4.2.4. to replace specific leakage frequencies for containment isolation with a reference to 10 CFR Part 50, Appendix J, Section III.D.3. RG&E requested these changes to the TS to support a one-time exemption from two parts of 10 CFR Part 50, Appendix J, Section III.D.3. These changes are administrative in nature.

The proposed language for TS 4.4.2.4.a is:

... individual penetrations and containment isolation valves shall be tested in accordance with 10 CFR Part 50, Appendix J, as modified by approved exemptions.

TS 4.4.2.4.a currently states:

... individual penetrations and containment isolation valves shall be tested during each reactor shutdown for refueling, or other convenient intervals, but in no case at intervals greater than 2 years.

The exemption to Appendix J, Section III.D.3 relieves RG&E from performing Type C tests during the 1995 refueling outage except for isolation valves which have maintenance performed on them or valves which have not demonstrated acceptable leakage during the previous two leakage rate tests. Second, the exemption relieves RG&E from the requirement to perform Type C tests within a 2-year interval. RG&E has been granted up to a 1-month extension of the 2-year interval for 129 containment isolation valves. Appendix J requires Type C testing for the 1995 refueling outage to be completed prior to exiting cold shutdown which is tentatively scheduled for April 27, 1995. RG&E will continue to perform Type B testing as required by Appendix J and approved exemptions, as well as the proposed TS, which reference Appendix J requirements.

In addition, the current TS permits Type C tests at "other convenient intervals." This contradicts Appendix J which requires Type C testing during reactor shutdowns for refueling. Thus, the TS contradicts the regulation and is not valid. RG&E's proposed change to the TSs fixes this problem.

2.0 EVALUATION

The TS do nothing more than reference the regulations pertaining to Type C testing. The TS do not need to duplicate specific terms in the regulations. The TS also encompass the recent Appendix J exemption referenced above.

The NRC staff finds that RG&E's proposal is acceptable for several reasons. As discussed in RG&E's March 15, 1995, letter, the performance of the containment isolation valves and the Ginna overall containment integrity have been good. The as-left Type A test leakage rate is 35% of L. The current Type B and C as-left maximum path leakage rate is 61% of the 0.6 L, Appendix J limit. Furthermore, RG&E has proposed to limit the exemption only to those valves on which no maintenance has been done and which have passed the last two consecutive Type C leakage rate tests. The staff has granted similar requests in the past. On February 2, 1994, the staff granted a similar exemption to the River Bend Station licensee, and by a letter dated April 29, 1987, the staff granted a similar request to the Washington Nuclear Power Plant, Unit 2 licensee.

As noted above, the present TS 4.4.2.4.a is inconsistent with Appendix J. RG&E has proposed to revise TS 4.4.2.4.a to make the wording consistent with Appendix J. the proposed TS is:

... individual penetrations and containment isolation valves shall be tested in accordance with 10 CFR Part 50, Appendix J, as modified by approved exemptions.

For the reasons set forth above, the NRC staff finds this TS change to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the

amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 15167). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Lobel

Date: April 26, 1995

April 26, 1995

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Allen R. Johnson, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

- Enclosures: 1. Amendment No. 59 to License No. DPR-18
- 2. Safety Evaluation

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