

Docket No.: 50-333

Mr. John C. Brons
Senior Vice President -
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. 102 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application dated October 9, 1984 (JPN-84) regarding reactor coolant leakage limits and surveillance.

The amendment revises the Technical Specifications to comply with the guidelines of Generic Letter 84-11 by imposing a more restrictive leakage limit as well as increased surveillance requirements.

Our review, however, has indicated that your amendment request does not incorporate the visual examination requirement following each containment decontamination, as specified in Attachment 1 to Generic Letter 84-11. To fully comply with the Generic Letter, we request that you submit an amendment to address this requirement, or provide justification as to why this item is not appropriate for the FitzPatrick facility.

A copy of the Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by

Harvey I. Abelson, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

- 1. Amendment No. 102 to License No. DPR-59
- 2. Safety Evaluation

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cc w/enclosures:
See next page

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Mr. John C. Brons
Power Authority of the State of New York

James A. FitzPatrick Nuclear
Power Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated October 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

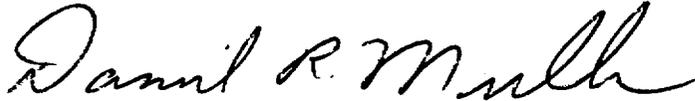
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: 10-31-86

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO DPR-59

DOCKET NO. 50-333

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages

141
141a*
142

*Page added

3.6 (cont'd)

4. Except as specified in 3.6.C.3 above, the reactor coolant water shall not exceed the following limits with steaming rates greater than or equal to 100,000 lb/hr and during reactor shutdowns.

Conductivity	5 μ mho/cm
Chloride ion	0.5 ppm

5. If Specification 3.6.C cannot be met, the reactor shall be placed in a cold condition within 24 hours.

D. Coolant Leakage

1. Anytime irradiated fuel is in the reactor vessel and the reactor coolant temperature is above 212°F, the reactor coolant leakage into the primary containment shall be limited to:
 - a. 5 gpm unidentified leakage
 - b. 2 gpm increase in unidentified leakage within any 24 hour period. (This limitation shall apply only after a period of 24 hours at operating pressure.)
 - c. The total reactor coolant leakage into the primary containment shall not exceed 25 gpm.
2. With any reactor coolant system leakage greater than any one of the limits specified in 3.6.D.1.a or 3.6.D.1.c above, the leakage rate shall be reduced to within these limits

4.6 (cont'd)

4.6.D

Coolant Leakage

Reactor coolant leakage rate inside the primary containment shall be monitored and recorded once every 4 hours utilizing the Primary Containment Sump Monitoring System (equipment drain sump monitoring and floor drain sump monitoring).

3.6 (cont'd)

4.5 (cont'd)

within 4 hours or the reactor shall be in at least the hot standby condition within the following 12 hours and in cold condition within the next 24 hours.

3. If the increase in unidentified leakage as specified in 3.6.D.1.b is exceeded, the source of the leakage shall be identified within 4 hours or the reactor shall be in at least hot standby condition within the next 12 hours and in cold condition within the following 24 hours.
4. The primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring and Floor Drain Sump Monitoring) and the Primary Containment Atmosphere Monitoring System (Gaseous and Particulate) shall be operable during reactor power operation.

3.6 (cont'd)

5. With the Primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring or Floor Drain Sump Monitoring) inoperable, restore the system to operable status within 24 hours or immediately initiate an orderly shutdown and be in at least hot standby condition within the next 12 hours and in cold condition within the following 24 hours.
6. With the Primary Containment Atmosphere Radioactivity Monitoring System (gaseous) or the Primary Containment Atmosphere Radioactivity Monitoring System (particulate) inoperable, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours. Otherwise be in at least hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours.

4.6 (cont'd)

3. Drywell Continuous Atmosphere Radioactivity Monitoring System instrumentation shall be functionally tested and calibrated as specified in Table 4.6.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

The Power Authority of the State of New York (the licensee) in their submittal dated October 9, 1984, proposed revisions to Section 3.6.D (Coolant Leakage) of the James A. FitzPatrick Technical Specifications (TS). These revisions imposed a more restrictive reactor coolant leakage limit and surveillance requirements, and were originally submitted in an amendment request dated September 28, 1981, based on guidelines specified in NUREG-0313, Revision 1. The revisions were subsequently updated as a result of an NRC request to comply with the guidelines in Generic Letter 84-11. The proposed TS changes include the following:

1. The increase of unidentified reactor coolant leakage within any 24 hour period shall not exceed 2 gpm.
2. The reactor coolant leakage rate shall be monitored and recorded every 4 hours.
3. The inoperability period of the primary containment sump monitoring system shall not exceed 24 hours.

2.0 EVALUATION

The proposed TS revisions impose a more restrictive reactor coolant leakage limit and require that more frequent surveillance of system leakage be performed. These changes are part of a defense in depth to assure leak before break for reactor coolant boundary piping and are consistent with the guidance specified in NUREG-0313, Revision 1 and in Attachment 1 to Generic Letter 84-11 for these specific items. However, one of the positions contained in Generic Letter 84-11 requires that visual examination of the reactor coolant piping be performed (for leakage) after each deinertment of the primary containment and, that this requirement be included in the TS. This requirement was not addressed in the licensee's submittal. We therefore request that the licensee provide an amendment to incorporate the visual examination requirement, or provide their justification why this item is not appropriate for the FitzPatrick facility.

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We conclude that the proposed changes to the Fitzpatrick TS are acceptable on the basis that they satisfy the criteria specified in Generic Letter 84-11. The requirement for visual examination of the reactor coolant piping following containment deinertment should be the subject of a separate action.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Koo

Dated: 10-31-86