

May 29, 1985

Docket No. 50-333

Mr. John C. Brons
Senior Vice President -
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your request dated January 30, 1985.

The amendment revises the Technical Specifications by changing Table 3.7-1, "Process Pipeline Penetrating Primary Containment," to correct an error concerning the isolation signals specified for two reactor water sample line valves.

A copy of our Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Harvey I. Abelson, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 91 to License No. DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. John C. Brons
Power Authority of the State of New York

James A. FitzPatrick Nuclear
Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated January 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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PDR ADOCK 050003333
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 29, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise the Technical Specifications by removing page 198 and inserting the revised page 198.

JAFNPP
TABLE J.7-1

PROCESS PIPELINE PENETRATING PRIMARY CONTAINMENT

(Numbers in parentheses are keyed to numbers on following page; equal codes are listed on following pages)

Line Isolated	Drywell Penetration	Valve Type (6)	Power to Open (5) (6)	Group	Location Ref. to Drywell	Power to Close (5) (6)	Isolation Signal	Closing Time (7)	Normal Status	Remarks and Exceptions
Main Steam Line	X-7A,B,C,D	AO Globe	Air and AC, DC	A	Inside	Air and spring	B,C,D,P,E	Note (1)	Open	
Main Steam Line	X-7A,B,C,D	AO Globe	Air and AC, DC	A	Outside	Air and spring	B,C,D,P,E	Note (1)	Open	
Main Steam Line Drain	X-8	MO Gate	AC	A	Inside	AC	B,C,D,P,E	15 sec	Closed	
Main Steam Line Drain	X-8	MO Gate	DC	A	Outside	DC	B,C,D,P,E	15 sec	Closed	
From Reactor Feedwater	X-9A, B	Check	-	A	Outside	Process	Rev. flow	NA	Open	
From Reactor Feedwater	X-9A, B	Check	-	A	Inside	Process	Rev. flow	NA	Open	
Reactor Water Sample	X-41	AO Globe	Air and AC	A	Inside	Spring	B,C	NA	Open	
Reactor Water Sample	X-41	AO Globe	Air and AC	A	Outside	Spring	B,C	NA	Open	
Control Rod Hydraulic Return	X-36	Check	-	A	Inside	Process	Rev. flow	NA)) Opens on Rod movement and closed at all other times, Note (4)	
Control Rod Hydraulic Return	X-36	Check	-	A	Outside	Process	Rev. flow	NA)		
Control Rod Drive Exhaust	X-38	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Exhaust	X-38	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Inlet	X-37	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		
Control Rod Drive Inlet	X-37	SO Valves	Air and AC	A	Outside	Spring	Note (4)	NA)		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 91 TO FACILITY OPERATING

LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 Introduction

By letter dated January 30, 1985, the Power Authority of the State of New York (PASNY), licensee for the James A. FitzPatrick Nuclear Power Plant, submitted proposed changes to the Technical Specifications (TS) to correct an error in Table 3.7-1, "Process Pipeline Penetrating Primary Containment," that was inadvertently introduced at the initial issuance of the TS. The proposed amendment changes isolation signals for two reactor water sample system (RWS) valves (drywell penetration X-41) from "B, C, D, E & P" to "B & C" to reflect the original design basis of the plant.

2.0 Evaluation

The isolation valves for the process lines communicating directly with the reactor vessel form isolation valve group A. Each of these lines has two isolation valves in series, one inside and one outside the primary containment. The two RWS isolation valves located on a 3/4" sample line are among this group.

Isolation signals B, C, D, E and P are part of an isolation function that effects closure of various group A isolation valves. For certain trip conditions, only certain isolation functions are initiated. Group A isolation functions include generation of isolation signals for the following components:

1. Main steam line isolation valves
2. Main steam line drain isolation valves
3. RWS isolation valves
4. Condenser vacuum pump

The condenser vacuum pump is tripped only for a main steam line high radiation trip. RWS isolation valves are designed to receive an isolation signal only for the following two signals:

- "B" "Reactor vessel low low water level"
- "C" "Main steam line high radiation"

The remaining group A valves are designed to isolate on any of the following signals:

- "B" "Reactor vessel low low water level"
- "C" "Main steam line high radiation"
- "D" "Steam line high flow"
- "E" "Steam tunnel high temperature"
- "P" "Main steam line low pressure"

Based on a review of the above information, the staff finds that the inclusion of signals D, E and P in Table 3.7-1 for drywell penetration X-41 does not reflect the actual design basis of the plant and represents an error.

The staff concludes that the proposed TS revision corrects an error to make the TS conform to the design basis of the plant and is, therefore, acceptable.

3.0 Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Salminen

Dated: May 29, 1985