

November 12, 1985

Docket No. 50-333

Mr. John C. Brons
Senior Vice President -
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. 96 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your request dated January 28, 1985 concerning your Inservice Inspection Program.

This amendment revises the Technical Specifications to comply with 10 CFR 50.55a(g)(4)(ii) and (5)(i).

A copy of the Safety Evaluation is also enclosed.

Sincerely,
Original signed by/

Harvey J. Abelson, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 96 to License No. DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. John C. Brons
Power Authority of the State of New York

James A. FitzPatrick Nuclear
Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated January 28, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages

144

153

157 - 162

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3.6 (cont'd)

F. Structural Integrity

The structural integrity of the Reactor Coolant System shall be maintained at the level required by the original acceptance standards throughout the life of the Plant.

G. Jet Pumps

Whenever the reactor is in the startup/hot standby or run modes, all jet pumps shall be operable. If it is determined that a jet pump is operable, the reactor shall be placed in a cold condition within 24 hours.

Amendment No. 96

4.6 (cont'd)

F. Structural Integrity

1. Nondestructive inspections shall be performed on the ASME Boiler and Pressure Vessel Code Class 1, 2 and 3 components and supports in accordance with the requirements of the weld and support inservice inspection program. This inservice inspection program is based on an NRC approved edition of, and addenda to, Section XI of the ASME Boiler and Pressure Vessel Code which is in effect 12 months or less prior to the beginning of the inspection interval.
2. An augmented inservice inspection program is required for those high stressed circumferential piping joints in the main steam and feedwater lines larger than 4 inches in diameter, where no restraint against pipe whip is provided. The augmented inservice inspection program shall consist of 100 percent inspection of these welds per inspection interval.

G. Jet Pumps

Whenever there is recirculation flow with the reactor in the startup/hot standby or run modes, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously:

3.6 and 4.6 BASES (cont'd)

not required to be operable (reactor coolant temperature 212°F and the reactor vessel vented or the reactor vessel head removed). Permitting physics testing and operator training under these conditions would not place the plant in an unsafe condition.

F. Structural Integrity

A pre-service inspection of the ASME Code Class 1 components was performed after site erection to assure the system was free of gross defects. An initial inspection program as detailed in Appendix F of the FSAR was developed and based on an approved edition of the ASME Code.

The program has been expanded to include the requirements of later, approved ASME Code editions and addenda as far as practicable. The importance of these inspections is recognized, and efforts to develop practical new alternative methods of assuring plant inservice integrity will continue. This inspection program should assure the detection of problem areas well before they represent a significant impact on safety.

Several locations on the main steam lines and feedwater lines are not restrained to prevent pipe whip in the event of pipe failure at these locations. The physical layout within the drywell precludes restraints at these points. Unrestrained high stress areas have been identified in these lines where breaks could result in pipe whip such that the pipe could impact the primary containment wall. Augmented inservice inspection of these weld locations shall be performed during each inspection period.

In addition, visual inspection in accordance with the approved ASME code will be made during periodic pressure and hydrostatic tests of critical systems. The inspection program specified encompasses the major areas of the vessel and piping system within the drywell. The inspection period is based on the observed rate of defect growth from fatigue studies sponsored by the AEC.

These studies show that thousands of stress cycles, at stresses beyond any expected to occur in a Reactor Coolant System, were required to propagate a crack. The test

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Amendment No. 96

157-162



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

In the Fall of 1976, the NRC staff sent guidance to all licensees of power reactors concerning the implementation of 10 CFR 50.55a(g), Inservice Inspection (ISI) Requirements. In that guidance, licensees were requested to submit, at least 6 months before the end of their first 120-month inspection interval, proposed Technical Specification (TS) changes to reflect more recent editions of Section XI of the ASME Boiler and Pressure Vessel Code, as required by 10 CFR 50.55a(g)(4)(ii) and (5)(i). These revisions to the inservice inspection program TS would allow the incorporation of improved examination techniques as well as expanded and more clearly defined sampling plans that may have been developed during the previous 120-month interval for pressure-retaining components and their supports.

2.0 EVALUATION

The staff has reviewed the licensee's request for amendment dated January 28, 1985. The proposed TS changes ensure that inservice inspection of components and piping will be performed in accordance with periodically updated editions and addenda of Section XI of the ASME Boiler and Pressure Vessel Code. The inspection and testing programs include ASME Code Class 1, 2 or 3 components and piping and will provide assurance that the structural integrity of these components and piping will be maintained at an acceptable level throughout the life of the plant. We conclude that the proposed changes conform to the above-mentioned staff guidance and, therefore, are acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria

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for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Turovlin

Dated: November 12, 1985