

August 22, 1983

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Docket No. 50-333

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Mr. J. P. Bayne
Executive Vice President
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Bayne:

SUBJECT: EXEMPTION REQUEST FROM CERTAIN REQUIREMENTS OF 10 CFR 50.44(c)
INVOLVING COMBUSTIBLE GAS CONTROL

Re: James A. FitzPatrick Nuclear Power Plant

The Commission has issued the enclosed Exemption from certain requirements of Section 50.44(c)(3)(ii) of 10 CFR Part 50 in response to your letter dated June 29, 1983, as supplemented by your letter dated July 19, 1983. This Exemption, which is being forwarded to the Office of the Federal Register for publication, permits an extension in the date of compliance with the requirements of 10 CFR 50.44(c)(3)(ii) until December 31, 1983.

In your letter you also requested permanent exemption from the requirements of 10 CFR 50.44(c)(3)(ii) based on BWR Owners Group studies of combustible gas control submitted for NRC review by letter dated June 21, 1982. We have very nearly completed our review of those studies, and will be able to consider your request following our completion of that review.

Sincerely,

Original signed by/

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

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PDR ADOCK 05000333
P PDR

Enclosure:
Exemption

cc w/enclosure:
See next page

OFFICE	ORB#2:DL	ORB#2:DL	C/ORB#2:DL	AD/ORB:DL	OELD	D:DL	DSI/CSB
SURNAME	S.Norris:km	J.Hegner	D.Vassallo	G.Lainas	D. Eisenhut		
DATE	8/11/83	8/11/83	8/15/83	8/15/83	8/16/83	8/19/83	8/22/83

Mr. J. P. Bayne
Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant

cc:

Mr. Charles M. Pratt
Assistant General Counsel
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

U. S. Environmental Protection
Agency
Region II Office
Regional Radiation Representative
26 Federal Plaza
New York, New York 10007

Mr. Corbin A. McNeill, Jr.
Resident Manager
James A. FitzPatrick Nuclear
Power Plant
P. O. Box 41
Lycoming, New York 13093

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Robert Burns
Manager - Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, N.Y. 10601

Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Mr. Leroy W. Sinclair, Pres.
PASNY
10 Columbus Circle
New York, N. Y. 10019

Mr. Jay Dunkleberger
Division of Policy Analysis and Planning
New York State Energy Office
Agency Building 2, Empire State Plaza
Albany, New York 12223

Resident Inspector
c/o U.S. NRC
P. O. Box 136
Lycoming, New York 13093

Mr. A. Klausmann
Vice President-Quality Assurance
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Mr. George Wilverding, Chairman
Safety Review Committee
Power Authority of the State
of New York
123 Main Street
White Plains, N.Y. 10601

Mr. M. C. Cosgrove
Quality Assurance Superintendent
James A. FitzPatrick Nuclear
Power Plant
P.O. Box 41
Lycoming, New York 13093

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter) Docket No. 50-333
)
 POWER AUTHORITY OF THE STATE)
 OF NEW YORK)
)
 (James A. FitzPatrick Nuclear)
 Power Plant))

EXEMPTION

I.

The Power Authority of the State of New York (the licensee) is the holder of Facility Operating License No. DPR-59 (the license) which authorizes operation of the James A. FitzPatrick Nuclear Power Plant located in Oswego County, New York, at a steady state reactor core power level not in excess of 2436 megawatts thermal. This license provides, among other things, that it is subject to all rules, regulations and Orders of the Commission now or hereafter in effect.

II.

On October 2, 1980, the Commission proposed rulemaking on "Interim Requirements Related to Hydrogen Control and Certain Degraded Core Considerations." The proposed amendments to 10 CFR Part 50 would improve hydrogen management in light-water reactor facilities and provide specific design and other requirements to mitigate the consequences of accidents.

On January 4, 1982, the proposed rule became effective and as part of the amendments, it required hydrogen recombiner capability to reduce the likelihood of venting radioactive gases following an accident. The hydrogen recombiner capability applies to light-water nuclear power reactors that rely upon purge/repressurization systems as the primary means of hydrogen control.

Section 50.44(c)(3)(ii) of 10 CFR Part 50 requires that by the end of the first scheduled outage after May 5, 1982 and of sufficient duration to permit required modifications, each light-water power reactor, that relies upon a purge/repressurization system as the primary means for controlling combustible gases following a Loss-of-Coolant Accident, shall be provided with either an internal recombiner or the capability to install an external recombiner following the start of an accident.

III.

In a June 29, 1983 submittal, as supplemented by letter dated July 19, 1983, the licensee requested an exemption from the requirement of Section 50.44(c)(3)(ii) for provision of either an internal recombiner or the capability to install an external recombiner following the start of an accident. The request was based on BWR Owners Group studies of combustible gas control submitted for NRC review by letter dated June 21, 1982. In the event that the Commission is unable to issue promptly its decision on request for exemption from the equipment requirements of 50.44(c)(3)(ii), the licensee requested an extension of the schedule requirements of 10 CFR 50.44(c)(3)(ii).

We have very nearly completed our review of the BWR Owners Group studies on which the licensee's exemption request was based. We will be able to consider the licensee's request for permanent exemption following completion of that review.

During the interim period, with respect to combustible gas control in the event of a loss-of-coolant accident, the FitzPatrick plant can use the existing containment atmosphere control systems, in conjunction with the standby gas treatment systems, to avoid unacceptable combustible gas

concentrations. The containment atmosphere system maintains an inert atmosphere during normal operation and the Containment Atmosphere Dilution (CAD) system is used to control combustible gas concentrations after an accident. By means of the CAD system, hydrogen and oxygen concentrations are monitored as nitrogen is added to the containment atmosphere to dilute combustible gases. In the unlikely prospect of high containment vessel pressure, the pressure may be relieved by venting through the standby gas treatment system. A detailed procedure has been developed by the licensee, with operating personnel trained to use these systems in the control of combustible gases. We find these means of combustible gas control acceptable for interim operation of the James A. FitzPatrick Nuclear Power Plant through December 31, 1983.

IV.

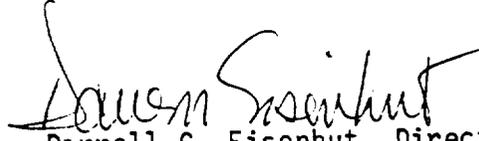
Accordingly, the Commission has determined that pursuant to 10 CFR 50.12, an exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest. Therefore, the Commission hereby approves the following exemption request.

Exemption is granted from the schedular requirement of Section 50.44 (c)(3)(ii) to extend the required date from "the end of the first scheduled outage beginning after July 5, 1982 and of sufficient duration to permit modifications" to no later than December 31, 1983, or, if the plant is shutdown on that date, before the resumption of operation thereafter.

The Commission has determined that the granting of this exemption will not result in any significant environmental impact and that, pursuant to

10 CFR 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland
this