May 20, 1996

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: ISSUANCE OF AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M94771)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated February 9, 1996.

This amendment changes the Technical Specification setpoints for the steam generator water level-high feedwater isolation function.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 63 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See attached sheet

DOCUMENT NAME: G:\GINNA\GI94771.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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cc w/encls: See next page

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DATED: May 20, 1996

AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR POWER PLANT

Docket File
PUBLIC
PDI-1 Reading
S. Varga, 14/E/4
J. Mitchell
S. Little
G. Vissing
H. Balukjian
OGC
G. Hill (2), T-5 C3
C. Grimes, 11/E/22
ACRS
PD plant-specific file
J. Linville, Region I

cc: Plant Service list

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated February 9, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Jocelyn A. Mitchell

Jocelyn A. Mitchell, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 20, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
3.3-27	3.3-27

Table 3.3.2-1 (page 3 of 3)
Engineered Safety Feature Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
5. Feedwater Isolation						
a. Automatic Actuation Logic and Actuation Relays	1,2[c],3[c]	2 trains	E,G	SR 3.3.2.7	NA	NA
b. SG Water Level – High	1,2[c],3[c]	3 per SG	F,G	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.5	≤ 94%	≤ 85%
c. Safety Injection	Refer to Function 1 (Safety Injection) for all initiation functions and requirements.					
6. Auxiliary Feedwater (AFW)						
a. Manual Initiation						
AFW	1,2,3	1 per pump	N	SR 3.3.2.4	NA	NA
Standby AFW	1,2,3	1 per pump	N	SR 3.3.2.4	NA	NA
b. Automatic Actuation Logic and Actuation Relays	1,2,3	2 trains	E,G	SR 3.3.2.7	NA	NA
c. SG Water Level – Low Low	1,2,3	3 per SG	D,G	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.5	≥ 16%	≥ 17%
d. Safety Injection (Motor driven pumps only)	Refer to Function 1 (Safety Injection) for all initiation functions and requirements.					
e. Undervoltage – Bus 11A and 11B (Turbine driven pump only)	1,2,3	2 per bus	D,G	SR 3.3.2.3 SR 3.3.2.5	≥ 2450 V with ≤ 3.6 sec time delay	≥ 2579 V with ≤ 3.6 sec time delay
f. Trip of Both Main Feedwater Pumps (Motor driven pumps only)	1,2	2 per MFW pump	B,C	SR 3.3.2.4	NA	NA

(c) Except when all Main Feedwater Regulating and associated bypass valves are closed and de-activated or isolated by a closed manual valve.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated February 9, 1996, the Rochester Gas and Electric Corporation (the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Technical Specifications (TSs). The requested changes would change the TS setpoints for the steam generator water level-high feedwater isolation function.

The following TS would be changed:

1. Technical Specification Table 3.3.2-1, Function 5b.
   - i. The allowable value would be changed from ≤68% to ≤94%.
   - ii. The trip setpoint value would be changed from ≤67% to ≤85%.

2.0 BACKGROUND

The licensee stated that the Steam Generator (SG) Water Level-High feedwater isolation setpoint is designed to prevent excessive moisture carryover to the main steam system, which would cause excessive wear on the main turbine. SGs separate water in two stages, a primary stage and a secondary stage. Primary separators are located in the steam drum of the generator below the secondary separators, and generally work on the centrifugal principle. The high water level setpoint is chosen such that water in the downcomer is not above the top of these primary separators which would flood them and degrade their performance.

The Ginna Station is planning to replace its SGs during the 1996 Refueling Outage. The original SGs are Westinghouse Model 44 SGs. The primary separator flood point for these generators is at approximately 75 percent narrow range water level. Consequently, a setpoint sufficiently below this value to allow for instrument and process measurement uncertainty is chosen for the current setpoint value of 67 percent narrow range.

Replacement SGs are being manufactured by Babcock and Wilcox International. The flood point for the primary separators on these generators is above

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100 percent narrow range level. Consequently, new setpoints sufficiently below 100 percent to account for process measurement uncertainty are chosen for an allowable value (94 percent), and instrument uncertainty for a setpoint value (85 percent). This expanded range allows the operator more time to restore level to nominal conditions using controlled means prior to initiation of feedwater isolation.

Prior to implementing this TS change the SGs must be replaced. No other hardware changes are required.

3.0 EVALUATION

The design of the replacement SGs is such that the primary steam separators are located at a higher elevation in the steam drum than in the existing SGs. The bottom of the actual separator is approximately 14 inches above the upper narrow range level tap. Therefore, it is acceptable to operate with water levels above 100 percent narrow range level without degrading separator performance. Since water level above 100 percent cannot be monitored, 100 percent is chosen as the limit.

The Upgraded Final Safety Analysis Report (UFSAR) accident analyses that model this setpoint (Section 15.1.6, Combined Steam-Generator Atmospheric Relief Valve and Feedwater Control Valve Failures) currently use a value of 100 percent narrow range level. Therefore, the accident analysis is not affected by this proposed change as the same 100 percent narrow range level value remains.

The SG Water-Level-High feedwater isolation function is designed to prevent excessive moisture carryover to the main steam system. The proposed setpoint change does not degrade the capability of the moisture separators and therefore this function is unchanged. The UFSAR accident analyses that model this function (UFSAR Section 15.1.6) use a value of 100 percent narrow range level. The revised trip setpoint (85 percent) and allowable value (94 percent) were raised from the previous values to minimize challenges and to avoid trips. They remain bounded by the accident analysis value of 100 percent. Therefore, we find these proposed changes to be acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 7558). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Balukjian

Date: May 20, 1996