October 11, 1984

Docket No. 50-333

Mr. J. P. Bayne
Executive Vice President,
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Bayne:

The Commission has issued the enclosed Amendment No. 84 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your request dated July 25, 1984.

The amendment revises the limiting conditions for operation and surveillance requirements contained in Appendix A of the Technical Specifications for the instrumentation and components associated with the actuation logic for the Automatic Depressurization System.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Harvey I. Abelson, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. 84 to
License No. DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. J. P. Bayne
Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Power Authority of the State of New York (the licensee) dated July 25, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 11, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise the Appendix "A" Technical Specifications as follows:

Remove	Insert
66	66
67	67
68	68
69	69
119	119

TABLE 3.2-2

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design For Both Trip Systems	Remarks
1	2	Reactor Low-Low Water Level	$\geq$ -38 in. indicated level ($\geq$ 126.5 in. above the top of active fuel)	4 HPCI & RCIC Inst. Channels	Initiates HPCI, RCIC & SGTS.
2	2	Reactor Low-Low-Low Water Level	$\geq$ -146.5 in. indicated level ($\geq$ 18 in. above the top of active fuel)	4 Core Spray & RHR Instrument Channels	Initiates Core Spray, LPCI, and Emergency Diesel Generators.
				4 ADS Instrument Channels	Initiates ADS in conjunction with confirmatory low level, 120 second time delay and (CI or Core Spray pump discharge pressure interlock if not inhibited by ADS override switches.
3	2	Reactor High Water Level	$\leq$ +58 in. indicated level ($\leq$ 222.5 in. above the top of active fuel)	2 Inst. Channels	Trips HPCI and RCIC Turbines.
4	1	Reactor Low Level (inside shroud)	$\geq$ +352 in. above vessel zero ($\geq$ 0 in. above the top of active fuel)	2 Inst. Channels	Prevents inadvertent operation of containment spray during accident condition.

TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Trip Systems	Remarks
5	2	Containment High Pressure	$1 < p < 2.7$ psig	4 Inst. Channels	Prevents inadvertent operation of containment spray during accident condition.
6	1	Confirmatory Low Level	$\geq$ 12.5 in. indicated level ($\geq$ 177 in. above the top of active fuel)	2 Inst. Channels	ADS Permissive in conjunction with Reactor Low-Low-Low Water Level.
7	2	High Drywell Pressure	$\leq$ 2.7 psig	HPCI Inst. Channels	Initiates Core Spray LPCI, HPCI & SGTS
8	2	Reactor Low Pressure	$\geq$ 450 psig	4 Inst. Channels	Permissive for opening Core Spray and LPCI Admission valves.

TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Trip Systems	Remarks
9	1	Reactor Low Pressure	$50 \leq p \leq 75$ psig	2 Inst. Channels	In conjunction with PCIS signal permits closure of RHR (LPCI) injection valves.
10					
11	THIS ITEM INTENTIONALLY BLANK				
12	1 (See Note 3)	Core Spray Pump Start Timer (each loop)	11 $\pm$ 0.6 sec.	1 Inst. Channel	Initiates starting of core spray pumps. (each loop)

TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Item No.	Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Trip Systems	Remarks
13	1 (See Note 3)	RHR Pump Start Timer			
		1st Pump (A Loop)	1.0 + 0.5 (-) 0 sec.	1 Inst. Channel	Starts 1st Pump (A Lo
		1st Pump (B Loop)	1.0 + 0.5 (-) 0 sec.	1 Inst. Channel	Starts 1st Pump (B Lo
		2nd Pump (A Loop)	6.0 ± 0.5 sec.	1 Inst. Channel	Starts 2nd Pump (A Lo
		2nd Pump (B Loop)	6.0 ± 0.5 sec.	1 Inst. Channel	Starts 2nd Pump (B Lo
14	1	Auto Blowdown Timer	120 sec ± 5 sec.	2 Inst. Channels	Initiates ADS, in conjunction with Low-Low-Low Reactor Water Level, and LPCI or Core Spray Pump discharge pressure interlock, if not inhibited by ADS override switches.
15	2	RHR (LPCI) Pump Discharge Pressure Interlock	125 psig ± 20 psig	4 Inst. Channels	Defers ADS actuation pending confirmation of low pressure core cooling system operation.

3.5 (cont'd)

D. Automatic Depressurization System (ADS)

1. The ADS shall be operable whenever the reactor pressure is greater than 100 psig, and irradiated fuel is in the reactor vessel and prior to reactor startup from a cold condition, except as specified below:

   a. From and after the date that one of the seven relief/safety valves of the ADS is made or found to be inoperable for any reason while it is required, continued reactor operation is permissible only during the succeeding 30 days unless repairs are made and provided that during such time the HPCI System is operable.

   b. From the time that more than one of the seven relief/safety valves of the ADS are made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding 24 hrs. unless repairs are made and provided, that

4.5 (cont'd)

D. Automatic Depressurization System (ADS)

1. Surveillance of the Automatic Depressurization System shall be performed during each operating cycle as follows:

   a. A simulated automatic initiation which opens all pilot valves.

   b. Manually open each relief/safety valve while bypassing steam to the condenser and observe a $\geq 10\%$ closure of the turbine bypass valves, to verify that the relief/safety valve has opened.

   c. A simulated automatic initiation which is inhibited by the override switches.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 84 TO FACILITY OPERATING

LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 Introduction

By a letter dated July 25, 1984, from J. P. Bayne to D. B. Vassallo, the Power Authority of the State of New York (PASNY/the licensee) requested changes to the Technical Specifications for the James A. FitzPatrick Nuclear Power Plant. The proposed changes are made in conjunction with a change to the actuation logic for the Automatic Depressurization System (ADS). The change to the ADS logic eliminates the high drywell pressure permissive for ADS actuation. This modification was approved by the staff in references 1 and 2.

2.0 Evaluation

The licensee has specified the following changes to the Technical Specifications:

1. A new section 4.5.D.1.c has been added that requires surveillance testing of the override switches once per operating cycle. This is consistent with the staff safety evaluation given in reference 2.

2. Four changes were made to Table 3.2-2 ("Instrumentation That Initiates or Controls the Core and Containment Systems"):

   a) On page 66, Item No. 2 (Reactor Low-Low-Low Water Level Trip Function) eliminates "...high drywell pressure..." from the remark associated with the ADS instrument channels and adds the phrase "if not inhibited by ADS override switches." This is appropriate since the plant modification removes the high drywell pressure permissive for ADS actuation, and adds override switches.

   b) On page 67, Item No. 6 (Confirmatory Low Level) the remarks entry is revised to include the phrase "in conjunction with reactor Low-Low-Low Water Level." This is a clarification only and does not reflect any plant modifications.

c) On page 68, Item No. 11 (High Drywell Pressure) has been deleted in its entirety. This is acceptable since this signal has been eliminated as a permissive for ADS actuation.

d) On page 69, Item No. 14 (Auto Blowdown Timer) the phrase "...high drywell pressure..." was deleted from the remarks entry to reflect the elimination of high drywell pressure as an ADS actuation permissive. In addition, the phrase "...if not inhibited by the ADS override switches" has been added to the remark to reflect the addition of these new switches.

3.0 Environmental Consideration

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 References

1. D. B. Vassallo to J. P. Bayne "Response to NUREG-0737, Item II.K.3.18 ADS Modification," dated June 5, 1984.

2. D. B. Vassallo to J. P. Bayne "NUREG-0737, Item II.K.3.18, ADS Logic Modifications" dated June 3, 1983.

Principal Contributor: T. Collins

Dated: October 11, 1984