

October 11, 1984

Docket No. 50-333

Mr. J. P. Bayne
Executive Vice President,
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Bayne:

The Commission has issued the enclosed Amendment No. 85 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your request dated November 18, 1981, as supplemented by your letter of February 4, 1982.

The amendment revises the Technical Specifications to reduce the scram surveillance frequency for the Control Rod Drive (CRD) system from 15% every 8 weeks to 10% every 16 weeks.

A copy of our Safety Evaluation is enclosed.

Sincerely,

Original signed by/

Harvey I. Abelson, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 85 to License No. DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. J. P. Bayne
Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. DPR-59

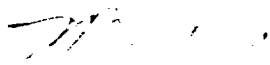
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated November 18, 1981, as supplemented by letter dated February 4, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 11, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise the Technical Specifications by removing and inserting the following page:

Remove

96

Insert

96

3.3 (cont'd)

2. The average of the scram insertion times for the three fastest operable control rods of all groups of four control rods in a two-by-two array shall be no greater than:

Control Rod Notch Position Observed	Average Scram Insertion Time (Sec.)
46	0.361
38	0.977
24	2.112
04	3.764

4.3 (cont'd)

2. At 16-week intervals, 10 percent of the operable control rod drives shall be scram timed above 950 psig. Whenever such scram time measurements are made, an evaluation shall be made to provide reasonable assurance that proper control rod drive performance is being maintained.
3. All control rods shall be determined operable once each operating cycle by demonstrating the scram discharge volume drain and vent valves operable when the scram test initiated by placing the mode switch in the SHUTDOWN position is performed as required by Table 4.1-1 and by verifying that the drain and vent valves:
- a. Close in less than 30 seconds after receipt of a signal for control rods to scram, and
 - b. Open when the scram signal is reset or the scram discharge instrument volume trip is bypassed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING

LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 Introduction

By letters dated November 18, 1981 and February 4, 1982, (References 1 and 2) the Power Authority of the State of New York (PASNY/the licensee) submitted a proposed Technical Specification revision which requested a reduction in the Control Rod Drive (CRD) scram surveillance frequency for the James A. FitzPatrick Nuclear Power Plant (FitzPatrick) from 15% every 8 weeks to 10% every 16 weeks.

2.0 Evaluation

Paragraph 4.3.2 of the FitzPatrick Technical Specifications requires the plant to perform a scram surveillance for its CRD at the frequency of 15% every 8 weeks. This frequency was established by Amendment No. 30 to the Technical Specifications issued on September 16, 1977. It was intended to protect the CRD mechanisms from possible accumulation of particles of corrosion products from carbon steel piping. However, a study performed by the General Electric Company (GE), forwarded as an attachment to Reference 2, has concluded that the presence of corrosion particles will not affect the reliability of the scram function of the CRD system. Using this conclusion as the basis, PASNY requested that the surveillance requirement be restored to the level prior to the September 16, 1977 amendment, i.e., 10% every 16 weeks.

The GE study evaluated the possible effects of corrosion particles generated from carbon steel piping of the flow stabilizer loop on the operation of the drive piston, the cooling water orifice, the ball check valve, and the possible effects of corrosion particles generated from the exhaust water header on the operation of the No. 121 directional control valves and associated filters. Results indicated the following:

1. The scram function of the CRD system will not be affected by the presence of corrosion particles. Should corrosion particles deposit at the seat of the ball valve, or in the directional valve filters, they will cause a change in flowrate, but the pressures and flow delivered to the drive during a rod scram are sufficient to compensate for this possibility.
2. Degradation of the CRD system caused by corrosion particles will be readily discovered during normal rod position change operations.
3. Frequent scram testing will not provide sufficient useful information about the condition of the CRD system, and may accelerate the system wear.

3.0 Summary

The staff agrees with these findings. The accumulation of corrosion particles will not prevent the CRD system from performing its normal scram function and minor system changes can be easily remedied by normal maintenance. The restoration of the CRD scram surveillance frequency to the level prior to the September 16, 1977 amendment is desirable and should be approved.

4.0 Environmental Consideration

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 References

1. Letter from J. P. Bayne to T. Ippolito, "James A. FitzPatrick Nuclear Power Plant Proposed Changes to the Technical Specifications" dated November 18, 1981.
2. Letter from J. P. Bayne to D. Vassallo, "Decreased Control Rod Drive Scram Surveillance Frequency" dated February 4, 1982.

Principal Contributor: H. Shaw

Dated: October 11, 1984