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Docket Nos. 50-286
and 50-333

Mr. George T. Berry, President
and Chief Operating Officer
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 32 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3 and Amendment No. 50 to Facility Operating License No. DPR-59 for the James A. Fitzpatrick Nuclear Power Plant. These amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated October 2, 1979 for both units, letters dated November 2, 1978, November 28, 1979 and May 23, 1980 for Indian Point 3 and letter dated December 6, 1979 for Fitzpatrick.

The amendments change the organization charts and position titles and increase the membership of the Safety Review Committee (SRC) for both units. In addition, the amendment for Indian Point 3 deletes a redundant review for the SRC, increases to 60 days the review by PORC and approval by the Resident Manager of temporary changes to environmental procedures, and changes the definition of "operable."

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,
Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

- Enclosures:
- 1. Amendment No. 32 to DPR-64
 - 2. Amendment No. 50 to DPR-59
 - 2. Safety Evaluation
 - 3. Notice of Issuance

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 5, 1980

Docket Nos. 50-286
and 50-333

Mr. George T. Berry, President
and Chief Operating Officer
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 32 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3 and Amendment No. 50 to Facility Operating License No. DPR-59 for the James A. Fitzpatrick Nuclear Power Plant. These amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated October 2, 1979 for both units, letters dated November 2, 1978, November 28, 1979 and May 23, 1980 for Indian Point 3 and letter dated December 6, 1979 for Fitzpatrick.

The amendments change the organization charts and position titles and increase the membership of the Safety Review Committee (SRC) for both units. In addition, the amendment for Indian Point 3 deletes a redundant review for the SRC, increases to 60 days the review by PORC and approval by the Resident Manager of temporary changes to environmental procedures, and changes the definition of "operable."

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in black ink, appearing to read "Steven A. Varga".

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 32 to DPR-64
2. Amendment No. 50 to DPR-59
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

Mr. George T. Berry
Power Authority of the State of New York - 2 - September 5, 1980

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Honorable George Begany
Mayor, Village of Buchanan
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Mr. George T. Berry
Power Authority of the State of New York - 3 - September 5, 1980

cc: Director, Technical Assessment Division
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT STATION UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 32
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Power Authority of the State of New York (the licensee) dated November 2, 1978, October 2, 1979, November 28, 1979 and May 23, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 32, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 5, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 32

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
1-2	1-2
6-2	6-2
6-7	6-7
6-8	6-8
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-13	6-13

Revise Appendix B as follows:

5.3	5.3
5.12	5.12

1.2.3 Reactor Critical

When the neutron chain reaction is self-sustaining and $k_{eff} = 1.0$.

1.2.4 Power Operation Condition

When the reactor is critical and the neutron flux power range instrumentation indicates greater than 2% of rated power.

1.2.5 Refueling Operation Condition

When the reactor is subcritical by at least 10% $\Delta k/k$ and T_{avg} is $\leq 140^{\circ}F$ and core alterations are being made with the head completely unbolted.

1.3 REFUELING OUTAGE

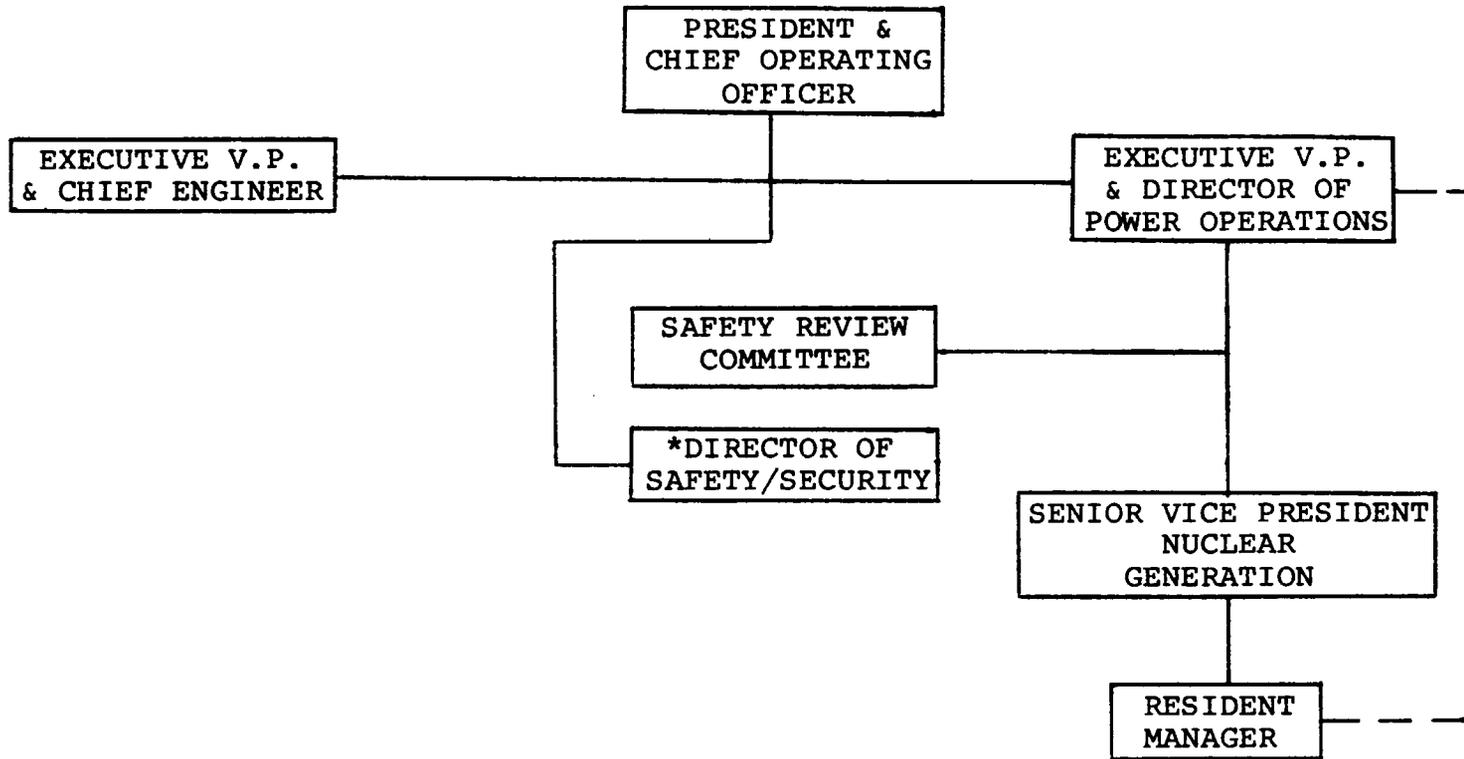
An outage in which core alterations are performed in order to compensate for fuel burnup.

1.4 CORE ALTERATION

The addition, removal, relocation or other movement of fuel, controls, or installed equipment or material in a reactor core, except for functions normally performed during conventional reactor operation in accordance with intended design of equipment, such as control rod or instrument detector movement or performance of flux scans.

1.5 OPERABLE

Properly installed in the system and capable of performing the intended functions in the intended manner as verified by testing and tested at the frequency required by the Technical Specifications. Implicit in this definition shall be the assumption that all necessary attendant controls, electrical power source, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).



*RESPONSIBLE FOR POWER AUTHORITY
DIRECTION OF FIRE PROTECTION PROGRAM

-----ADMINISTRATIVE

_____FUNCTIONAL

FIGURE 6.2-1
MANAGEMENT ORGANIZATION CHART
INDIAN POINT 3 NUCLEAR POWER PLANT

- c. Review of all proposed changes to the Operating License and Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Resident Manager, who will forward the report to the Chairman of the Safety Review Committee and Senior Vice President-Nuclear Generation.
- f. Review of events requiring 24 hour notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Resident Manager or the Chairman of the Safety Review Committee (SRC).
- i. Review of the Plant Security Plan and implementing procedures annually.
- j. Review of the Emergency Plan and implementing procedures annually.

AUTHORITY

6.5.1.7 The Plant Operating Review Committee shall:

- a. Recommend to the Resident Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide notification within 24 hours to the Chairman of the SRC and the Senior Vice President-Nuclear Generation of disagreement between the PORC and the Resident Manager; however the Resident Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and Senior Vice President-Nuclear Generation.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall collectively have the competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Other appropriate fields associated with the unique characteristics of a nuclear power plant.
- m. Licensing

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Chairman:	Director-Nuclear Design and Analysis
Vice-Chairman:	Director-Quality Assurance
Member:	Manager-Nuclear Technical Support
Member:	Radiological Engineer-Design and Analysis (Secretary of SRC)
Member:	Senior Vice President-Nuclear Generation
Member:	Director-Electrical Design and Analysis
Member:	Director of Environmental Programs
Member:	Director-Civil/Structural Design and Analysis
Member:	Senior Inservice Inspection Engineer-Nuclear Technical Support
Member:	Manager-Nuclear Licensing
Member:	Senior Nuclear Engineer-Design and Analysis
Member:	Director-Mechanical Design and Analysis

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and five members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hours written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the Executive Vice President.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. A fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Executive Vice President and Director of Power Operations (to be referred to in the Tech Spec. as Executive Vice President) on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Executive Vice President within 14 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Executive Vice President within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHARTER

6.5.2.11 Conduct of the committee will be in accordance with a charter, approved by the Executive Vice President, setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hours notification to the Commission shall be reviewed by the PORC and a report submitted by the Resident Manager to the Chairman of the SRC and Senior Vice President-Nuclear Generation.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36(c)(1)(i).
- b. The Safety Limit violation shall be reported immediately to the Commission. The Chairman of the SRC and Senior Vice President-Nuclear Generation will be notified within 24 hours.

- c. A Safety Limit Violation Report shall be prepared by the PORC. This report shall describe (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Chairman of the SRC and the Senior Vice President-Nuclear Generation by the Resident Manager.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.

6.8.2 Temporary changes to procedures above may be made provided:

- a. The intent of the original procedures is not altered.
- b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.
- c. The change is documented, reviewed by the PORC and approved by the Resident Manager within 14 days of implementation.

6.8.3 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PORC and approved by the Resident Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

- c. Review proposed changes in the Operating License and Technical Specifications relating to environmental concerns.
- d. Make or cause to be made periodic audits of plant operation to verify conformance with the Environmental Technical Specifications.
- e. Review violations of the Environmental Technical Specifications.

5.5 PROCEDURES

5.5.1 Detailed written procedures, including applicable checklists and instructions, shall be prepared and followed for all activities involved in carrying out the environmental monitoring program. Procedures include sampling, data recording and storage, instrument calibration, measurements and analyses, and actions to be taken when limits are approached or exceeded. Testing frequency of alarms, as determined from experience with similar instruments in similar environments and from manufacturers' technical manuals, have also been included.

5.5.2 Plant Operating Procedures include provisions, in addition to the procedures specified in Section 5.5.1, to ensure that all plant systems and components are operated in compliance with the limiting conditions for operations established as part of the Environmental Technical Specifications.

- 5.3.3 Temporary changes to non-radiological environmental procedures above may be made provided:
- a. The intent of the original procedures is not altered.
 - b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.
 - c. The change is documented, reviewed by the PORC and approved by the Resident Manager within 60 days of implementation.

5.6 PLANT REPORTING REQUIREMENTS

- 5.6.1 Routine Reports
- a. Annual Environmental Operating Report

Part A: Nonradiological Report. A report on the environmental surveillance programs for the previous 12 months of operation shall be submitted to the Director of the NRC Regional Office (with a copy to the Director, Office of Nuclear Reactor Regulation) as a separate document within 120 days after January 1 of each year. The report shall include summaries, interpretations, and statistical evaluation of the results of the nonradiological environmental surveillance activities (Section 3) and the environmental monitoring programs required by limiting conditions for operation (Section 2) for the report period, including a comparison with preoperational studies, operational controls (as appropriate), and previous environmental surveillance

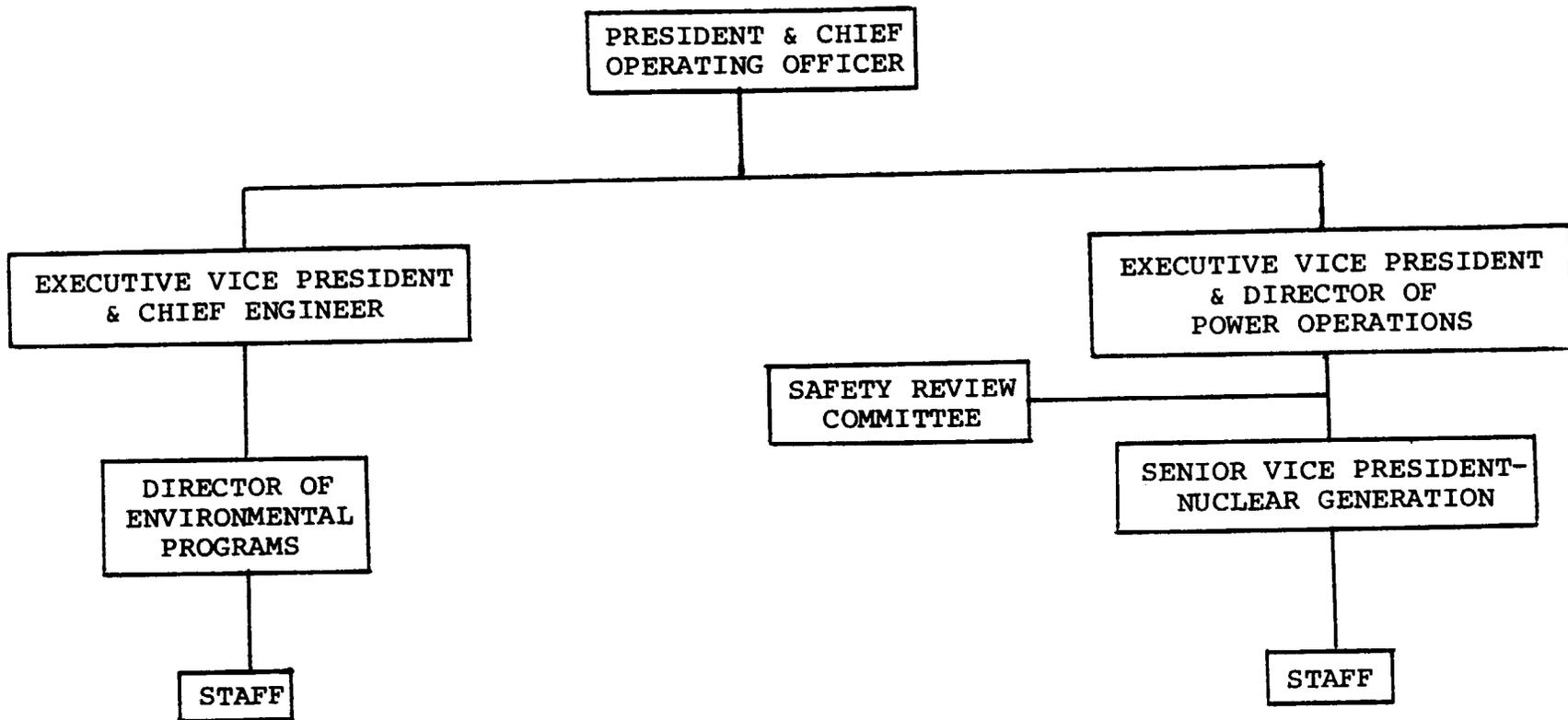


FIGURE 5.2-2
 MANAGEMENT ORGANIZATION ENVIRONMENTAL
 INDIAN POINT 3 NUCLEAR POWER PLANT



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Power Authority of the State of New York (the licensee) dated October 2, 1979 and December 6, 1979 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

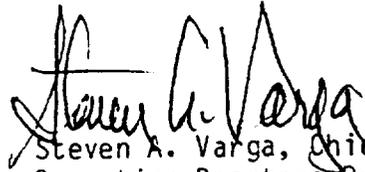
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 5, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
247	247
249	249
250	250
251	251
252	252
	252a
253	253
259	259

Replace page 44 of the Appendix B Technical Specifications with the enclosed page 44.

6.0

ADMINISTRATIVE CONTROLS

Administrative Controls are the means by which plant operations are subject to management control. Measures specified in this section provide for the assignment of responsibilities, plant organization, staffing qualifications and related requirements, review and audit mechanisms, procedural controls and reporting requirements. Each of these measures are necessary to ensure safe and efficient facility operation.

6.1

RESPONSIBILITY

The Resident Manager is responsible for safe operation of the plant. During periods when the Resident Manager is unavailable, the Superintendent of Power will assume his responsibilities. In the event both are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel. The Resident Manager reports directly to the Executive V.P. and Director of Power Operations for administrative matters and functionally to the Sr. Vice President Nuclear Generation for operational related matters, as shown in Fig. 6.1-1.

6.2

PLANT STAFF ORGANIZATION

The plant staff organization is shown graphically in Fig. 6.2-1 and functions as follows:

1. A licensed senior reactor operator shall be on site at all times when there is fuel in the reactor.
2. In addition to item 1 above, a licensed reactor operator shall be in the control room at all times when there is fuel in the reactor.
3. In addition to items 1 & 2 above, a licensed reactor operator shall be readily available on site whenever the reactor is in other than cold condition.
4. Two licensed reactor operators shall be in the control room during start-ups and scheduled shutdowns.
5. A licensed senior reactor operator shall be responsible for all movement of new and irradiated fuel within the site boundary. A licensed reactor operator will be required to manipulate or directly supervise the manipulation of the controls of all fuel moving equipment, except the reactor building crane. All fuel movements by the reactor building crane, except new fuel movements from receipt through dry storage, shall be under the direct supervision of a licensed reactor operator. All fuel movements within the core shall be directly monitored by a member of the reactor analyst group. (a)

Superintendent and Reactor Analyst. Special consultant to provide expert advice may be utilized when the nature of a particular problem dictates.

(B) Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

(C) Meeting Frequency

Meetings will be called by the Chairman as the occasions for review or investigation arise. Meetings will be no less frequent than once a month.

(D) Quorum

The Chairman or Vice Chairman and four members, including designated alternates, shall constitute a quorum.

(E) Responsibilities

1. Review plant procedures, and changes thereto, required by Specification 6.8.
2. Review proposed tests and experiments that affect nuclear safety.
3. Review proposed changes to the Operating License and Technical Specifications.
4. Review proposed changes or modifications to plant systems or equipment that affect nuclear safety.
5. Investigate violations of the Technical Specifications and prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Resident Manager, who will forward the report to the Senior Vice President - Nuclear Generation and to the Chairman of the Safety Review Committee.
6. Review plant operations to detect potential safety hazards.
7. Review the Security Plan and implementing procedures annually.

8. Review the Emergency Plan and implementing procedures annually.
9. Perform special review and/or investigations at the request of the Resident Manager.
10. Review of those reportable occurrences requiring 24 hour notification to the NRC, in accordance with Specification 6.9.

(F) Authority

The PORC shall function to advise the Resident Manager on all matters related to nuclear safety and environmental operations. The PORC shall recommend approval or disapproval to the Resident Manager of those items considered in 6.5 1E (1) through (4) and determine if items considered in 6.5 1E (1) through (5) constitute unreviewed safety questions, as defined in 10 CFR 50.59.

In the event of a disagreement between the PORC and the Resident Manager, the Chairman of the SRC and the Sr. Vice President - Nuclear Generation, or their designated alternates, shall be notified within 24 hours and written notification provided on the next business day; however, the Resident Manager shall have responsibility for resolution of such disagreement pursuant to Section 6.1.

(G) Records

Minutes of all meetings of the PORC shall be recorded and numbered. Copies will be retained in file. Copies will be forwarded to the Chairman of the SRC and the Sr. Vice President - Nuclear Generation.

(H) Procedures

Conduct of the PORC and the mechanism for implementation of its responsibilities and authority are defined in the pertinent Administrative Procedures.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall collectively have the competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control

- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Other appropriate fields associated with the unique characteristics of a nuclear power plant
- m. Licensing

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

- Chairman: Director-Nuclear Design and Analysis
- Vice-Chairman: Director-Quality Assurance
- Member: Manager-Nuclear Technical Support
- Member: Radiological Engineer-Design and Analysis (Secretary of SRC)
- Member: Senior Vice President-Nuclear Generation
- Member: Director-Electrical Design and Analysis
- Member: Director of Environmental Programs
- Member: Director-Civil/Structural Design and Analysis
- Member: Director-Mechanical Design and Analysis
- Member: Senior Inservice Inspection Engineer-Nuclear Technical Support
- Member: Manager-Nuclear Licensing
- Member: Senior Nuclear Engineer-Design and Analysis

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and five members, including alternates. No more than minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluation for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of code, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDIT

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the President.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. An independent fire protection and loss of prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Executive Vice President and Director of Power Operations on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Executive Vice President within 14 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the President within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHARTER

6.5.2.11 Conduct of the committee will be in accordance with a charter approved by the Executive Vice President setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

(A) In the event of a Reportable Occurrence, the NRC shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

(B) Each Reportable Occurrence requiring 24 hours notification to the NRC shall be reviewed timely by the PORC and a report submitted by the Resident Manager to the Senior Vice President-Nuclear Generation and the SRC.

6.7 SAFETY LIMIT VIOLATION

(A) If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36 (c) (i).

(B) An immediate report of each safety limit violation shall be made to the NRC by the Resident Manager. The Senior Vice President-Nuclear Generation and Chairman of the SRC will be notified within 24 hours.

(C) The PORC shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluation of: (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility components systems or structures and (3) corrective action required to prevent recurrence. The Resident Manager shall forward this report to the Senior Vice President-Nuclear Generation, Chairman of the SRC and the NRC.

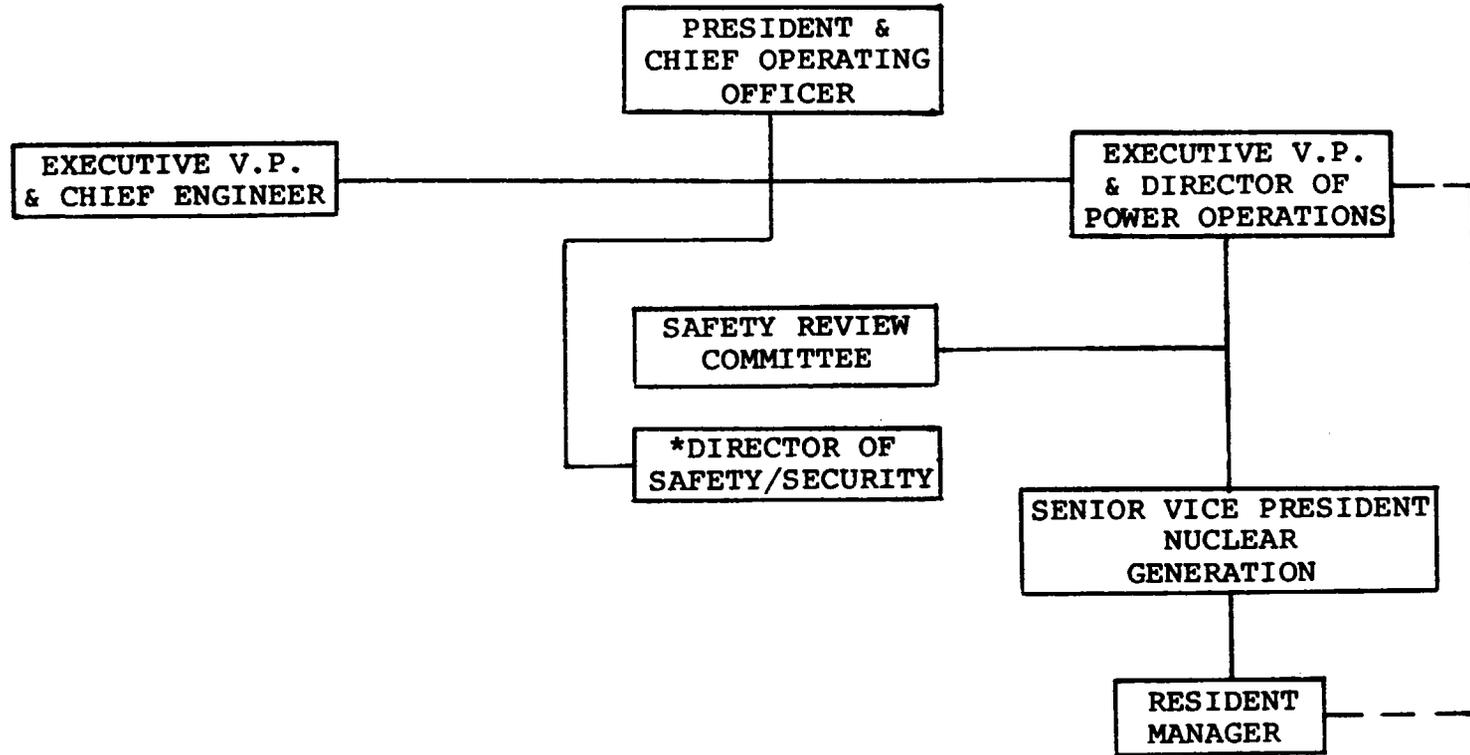
6.8 PROCEDURES

(A) Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5 "Facility Administrative Policies and Procedures" of ANSI 18.7-1972 and Appendix A of Regulatory Guide 1.33, November 1972. In addition, procedures shall be established, implemented and maintained for the Fire Protection Program.

(B) Those procedures affecting nuclear safety shall be reviewed by PORC and approved by the Resident Manager prior to implementation.

(C) Temporary changes to nuclear related procedures may be made provided:

1. The intent of the original procedure is not altered.



*RESPONSIBLE FOR POWER AUTHORITY
DIRECTION OF FIRE PROTECTION PROGRAM

-----ADMINISTRATIVE

_____FUNCTIONAL

FIGURE 6.1-1
MANAGEMENT ORGANIZATION CHART
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

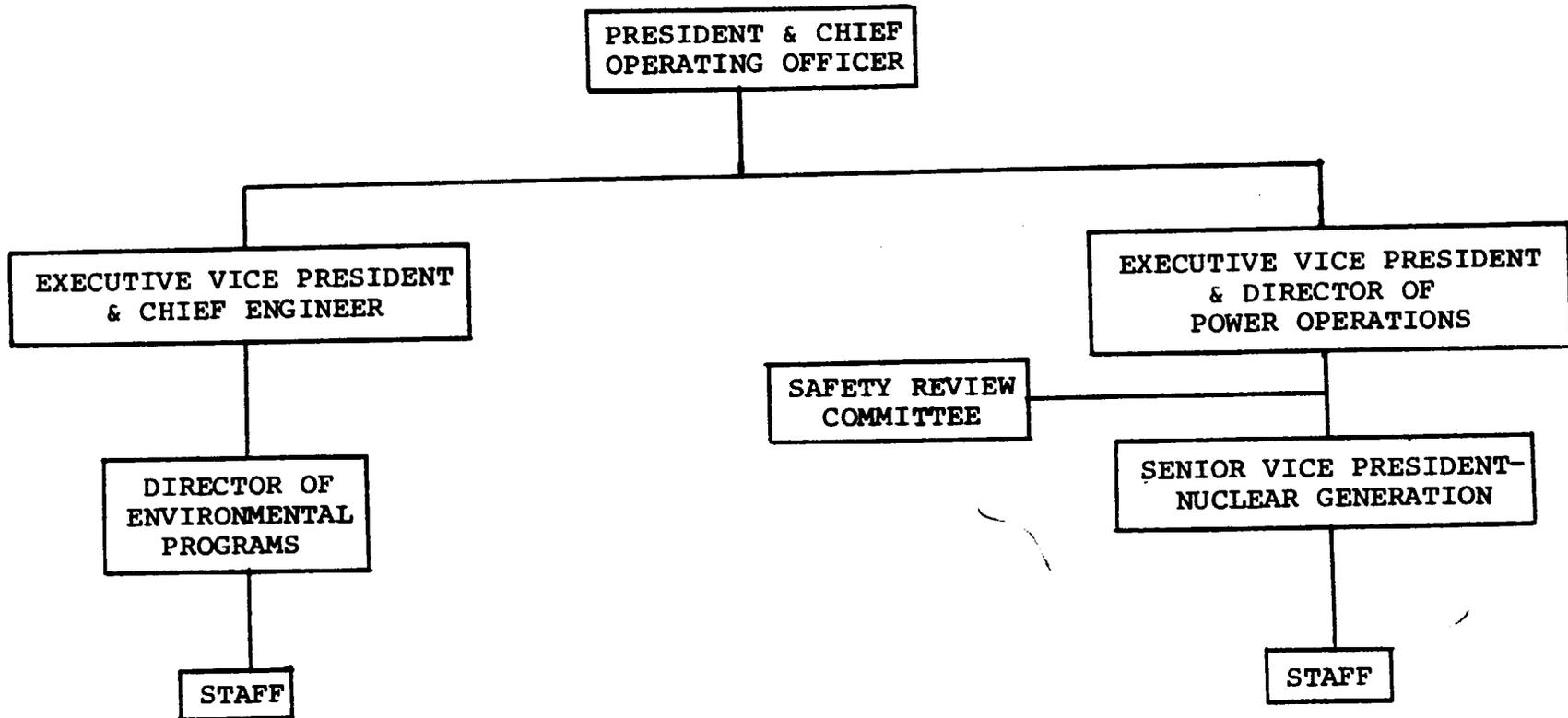


FIGURE 5.2-2
MANAGEMENT ORGANIZATION ENVIRONMENTAL
JAMES A. FITZPATRICK NUCLEAR POWER PLANT



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 32 TO FACILITY OPERATING LICENSE NO. DPR-64
AND AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NOS. 50-286 AND 50-333

Introduction

By letters dated November 2, 1978, October 2, 1979, November 28, 1979 and May 23, 1980 the Power Authority of the State of New York (the licensee) requested amendment of Facility Operating License No. DPR-64 for Indian Point Nuclear Generating Unit No. 3. By letters dated October 2, 1979 and December 6, 1979 the licensee requested amendment of Facility Operating License No. DPR-59 for the James A. Fitzpatrick Nuclear Power Plant.

The October 2, 1979 letters proposed minor changes to the organization charts and position titles that apply to both facilities. Additional position title changes were identified in a letter dated April 25, 1980 and subsequent discussions with the licensee.

The November 28, 1979 letter for Indian Point 3 and the December 6, 1979 letter for Fitzpatrick proposed an increase in the membership of the Safety Review Committee (SRC). In addition, the Indian Point 3 letter proposed to delete a redundant SRC review and the Fitzpatrick letter proposed to alter the format of Section 6.5.2 to make it consistent with the General Electric Standard Boiling Water Reactors Technical Specifications (GE STS).

The November 2, 1978 letter for Indian Point 3 proposed to increase to 60 days the review by the Plant Operating Review Committee (PORC) and approval by the Resident Manager of temporary changes to non-radiological environmental procedures.

The May 23, 1980 letter for Indian Point 3 responded to our April 10, 1980 request and proposed a change to the definition of the term "operable."

Evaluation

The proposed change to the organization chart has the Resident Managers reporting administratively to the Executive Vice President and Director of Operations instead of to the General Manager and Chief Engineer. Since the Executive Vice President and Director of Power Operations reports directly to the President and Chief Operating Officer, the proposed change will strengthen the licensee's management of nuclear operations. We, therefore, find this change acceptable. We also find acceptable changes in position titles.

The proposed increase in membership of the SRC will increase its expertise and diversity and is therefore acceptable. The Indian Point 3 Technical Specifications required the SRC to both review and audit the Fire Protection Program which is redundant. The proposed change deletes the review function while the audit function remains unchanged. This is consistent with the Standard Technical Specifications and, therefore, we find it acceptable. The change to the GE STS format in the Fitzpatrick proposal is also acceptable.

The proposal to increase to 60 days the PORC review and Resident Manager approval of temporary changes to the non-radiological environmental procedures makes the Technical Specifications consistent with Consolidated Edison's Technical Specifications for Indian Point, Unit Nos. 1 and 2. Since most of the non-radiological environmental procedures are the same for all three units, and are reviewed and approved jointly by PASNY and Consolidated Edison, we find this change acceptable.

The proposed definition of the term "operable" meets the intent of our April 10, 1980 letter to all power reactor licensees (Section 3.0.5 of the Model Technical Specifications). The other requirements in our April 10, 1980 letter are already satisfied in the Indian Point 3 Technical Specifications. We, therefore, conclude that the proposed change satisfactorily responds to our request and is acceptable.

Environmental Consideration

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that these amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because these amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, these amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 5, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-286 AND 50-333POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 32 to Facility Operating License No. DPR-64 issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) located in Buchanan, Westchester County, New York, and Amendment No. 50 to Operating License No. DPR-59, issued to the Power Authority of the State of New York, which revised the Technical Specifications for operation of the James A. Fitzpatrick Nuclear Plant (the facility) located in Oswego County, New York. These amendments are effective as of the date of issuance.

The amendments change the organization charts and position titles and increase the membership of the Safety Review Committee (SRC) for both units. In addition, the amendment for Indian Point 3 deletes a redundant review for the SRC, increases to 60 days the review by the Plant Operating Review Committee (PORC) and approval by the Resident Manager of temporary changes to environmental procedures, and changes the definition of "operable."

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has

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made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated November 2, 1978, October 2, 1979, November 28, 1979 and May 23, 1980, (2) Amendment No. 32 to License No. DPR-64 for Indian Point Unit 3, (3) Amendment No. 50 to License No. DPR-59 for the James A. Fitzpatrick Nuclear Power Plant, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C.; at the White Plains Public Library, 100 Martine Avenue, White Plains, New York for Indian Point 3; and at the Penfield Library, State University College at Oswego, Oswego, New York 13126 for Fitzpatrick.

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A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 5th day of September, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing