

January 30, 1985

Docket No. 50-333

Mr. C. A. McNeill
Executive Vice President,
Nuclear Generation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Dear Mr. McNeill:

The Commission has issued the enclosed Amendment No. 86 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment authorizes changes to the Technical Specifications in response to your request dated October 9, 1984 regarding a typographical error found in Table 3.2-7 of Appendix A.

The amendment revises the Technical Specifications by changing the high reactor pressure setpoint for recirculation pump trip from "greater than or equal to 1120 psig" to the corrected value of "less than or equal to

A copy of our Safety Evaluation is enclosed.

Sincerely,

Original signed by/

Harvey I. Abelson, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 86 to License No. DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. C. A. McNeill, Jr.
Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant .

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated October 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 86, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 30, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 86

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise the Technical Specifications by removing and inserting the following pages:

Remove

77

Insert

77

JAFNPP

TABLE 3.2-7

INSTRUMENTATION THAT INITIATES RECIRCULATION PUMP TRIP

<u>Minimum Number of Operable Instrument Channels per trip System (1)</u>	<u>Instrument</u>	<u>Trip Level Setting</u>	<u>Total Number of Instrument Channels Provided by Design for Both Channels</u>	<u>Action</u>
1	Reactor High Pressure	\leq 1120 psig	4	(2)
1	Reactor Low-Low Water Level	$>$ -38 in. indicated level (\geq 126.5 in. above the top of active fuel)	4	(2)

Notes for Table 3.2-7

1. Whenever the reactor is in the run mode, there shall be one operable trip system for each parameter for each operating recirculation pump. From and after the time it is found that this cannot be met, the indicated action shall be taken.
2. Reduce power and place the Mode Selector Switch in a Mode other than the Run Mode within 24 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 86 TO FACILITY OPERATING

LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 Introduction

By letter dated October 9, 1984, the Power Authority of the State of New York (the licensee) proposed certain changes to the Technical Specifications (TS) to correct a typographical error found in Table 3.2-7 of Appendix A.

As the existing table is written, instruments used to detect high reactor pressure for the purpose of initiating a recirculation pump trip are required to have a setpoint "greater than or equal to 1120 psid" rather than a setpoint "less than or equal to 1120 psig." This incorrectly establishes a lower limit instead of an upper limit for this setpoint.

2.0 Evaluation

The intent of a recirculation pump trip is to provide a means of reducing reactor power in the event of a failure-to-scam. Table 3.2-7 should correctly establish an upper limit for the reactor pressure trip setpoint; exceeding this setpoint should trip the recirculation pumps, thereby reducing reactor power and, consequently, reactor pressure.

In the Final Safety Analysis Report, Appendix E, Section E.32, the licensee, with reference to General Electric topical report NEDO-10349, "Analysis of Anticipated Transients Without Scram," committed to making design changes which would provide for tripping the recirculation pumps on a high reactor pressure event. The NEDO-10349 report concluded that tripping the recirculation pumps when reactor pressure was greater than or equal to 1150 psig was effective in keeping reactor power, pressures, and temperatures below safety limits and for allowing time for appropriate operator action. The licensee had historically used a setpoint value of 1135 psig until discovery of the error in August 1984. Shortly after discovery, the setpoint was readjusted to a value below 1120 psig.

Although the licensee failed to recognize the error in the TS until August 1984 we find that no significant safety hazard existed prior to its discovery and subsequent readjustment of the setpoint because plant procedures assured that at no time was the 1150 psig setpoint upper limit referenced in NEDO-10349 exceeded.

We conclude that the licensee's proposed amendment provides a necessary correction to the TS and is, therefore, acceptable.

3.0 Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Abelson

Dated: January 30, 1985