US MUCLEAR REGULATORY COMMISSION DOCUMENT MANAGEMENT BR

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Docket No. 50-333

Mr. George T. Berry President and Chief Operating Officer Power Authority of the State of New York 10 Columbus Circle New York, New York 10019

Dear Mr. Berry:

NRC FORM 318 (10/80) NRCM 0240

James A. FitzPatrick Nuclear Power Plant

The Commission has issued the enclosed Amendment No. 65 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. This amendment consists of changes to the Technical Specifications in response to your application dated January 6, 1982.

The amendment revises the provisions in the Technical Specifications to allow the reorganization of the FitzPatrick Safety Review Committee. In essence, personnel titles and reporting requirements have been changed in order to clarify responsibilities. Using various tools, such as the Task Action Plan, Regulatory Guide 1.33 and ANSI 18.7/ANS 3.2 guidelines, we reviewed the James A. FitzPatrick management organization to assure that the proposed change does not diminish the level of safety with respect to organization and technical resources provided by the existing Technical Specifications.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR \$51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

The amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and, therefore, does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

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OFFICIAL RECORD COPY

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Domenic B. Vassallo, Chief Operating Reactors Branch #2 Division of Licensing

Enclosures:

Amendment No. 65 to DPR-59

Notice

cc w/enclosures See next page

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cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65 License No. DPR-59

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated January 6, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (1) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief

Operating Reactors Branch #2
Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: February 3, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 65

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

1. Revise Appendix A Technical Specifications as follows:

Insert		
251 252 252a 253 259		

2. Revise Appendix B Technical Specifications by removing page 44 and inserting revised page 44.

- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- K. Civil/Structural Engineering
- 1. Nuclear Licensing
- m. Emergency Planning
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Member: Member: Member: Member: Member:	Manager-Nuclear Safety Evaluation Vice President-Quality Assurance Vice President Nuclear Support-BWR Vice President Nuclear Support-PWR Manager Radiological Health and Chemistry Director-Nuclear Design and Analysis Director-Electrical Design and Analysis Director of Environmental Programs Director-Civil/Structural Design and Analysis
Member:	Director-Civil/Structural Design and Analysis Director-Mechanical Design and Analysis
	Director-Mechanical Design and Analysis
Member:	Director-Piping and Process-Design and Analysis
Member:	Manager Operational Analysis and Training

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and four members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW .

6.5.2.7 The SRC shall review:

- a. The safety evaluation for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of code, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDIT

- 6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:
 - a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
 - b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the Senior Vice President-Nuclear Generation.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. An independent fire protection and loss of prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Senior Vice President-Nuclear Generation on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8

RECORDS

- 6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:
 - a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Generation within 14 days after the date of the meeting.
 - b. Reports of review encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President-Nuclear-Generation within 14 days following completion of the review.
 - shall be forwarded to the Senior Vice President-Nuclear Generation and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHARTER

6.5.2.11 Conduct of the committee will be in accordance with a charter approved by the Senior Vice President-Nuclear Generation setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

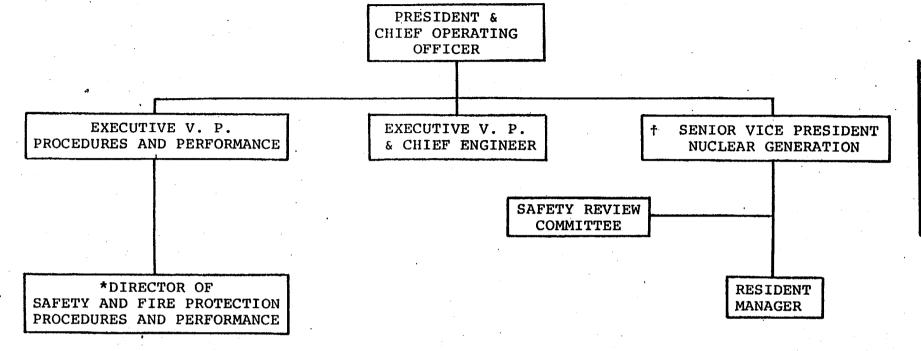
- (A) In the event of a Reportable Occurrence, the NRC shall be notified and or a report submitted pursuant to the requirements of Specification 6.9.
- (B) Each Reportable Occurrence requiring 24 hours notification to the NRC shall be reviewed timely by the PORC and a report submitted by the Resident Manager to the Senior Vice President-Nuclear Generation and the SRC.

6.7 SAFETY LIMIT VIOLATION

- (A) If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36 (c) (i).
- (B) An immediate report of each safety limit violation shall be made to the NRC by the Resident Manager. The Senior Vice President-Nuclear Generation and Chairman of the SRC will be notified within 24 hours.
- (C) The PORC shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluation of: (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility component systems or structures and (3) corrective action required to prevent recurrance. The Resident Manager shall forward this report to the Senior Vice President-Nuclear Generation, Chairman of the SRC and the NRC.

6.8 PROCEDURES

- (A) Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5 "Facility Administrative Policies and Procedures" of ANSI 18.7-1972 and Appendix A of Regulatory Guide 1.33, November 1972. In addition, procedures shall be established, implemented and maintained for the Fire Protection Program.
- (B) Those procedures affecting nuclear safety shall be reviewed by PORC and approved by the Resident Manager prior to implementation.
- (C) Temporary changes to nuclear related procedures may be made provided:
- 1. The intent of the original procedure is not altered.



The Senior Vice President Nuclear Generation is responsible for administerting the fire protection program.

*The Director of Safety and Fire Protection is the off-site management position responsible for assessing the effectiveness of the fire protection programs by conducting reviews and audits.

FIGURE 6.1-1

MANACEMENT ORGANIZATION CHART
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

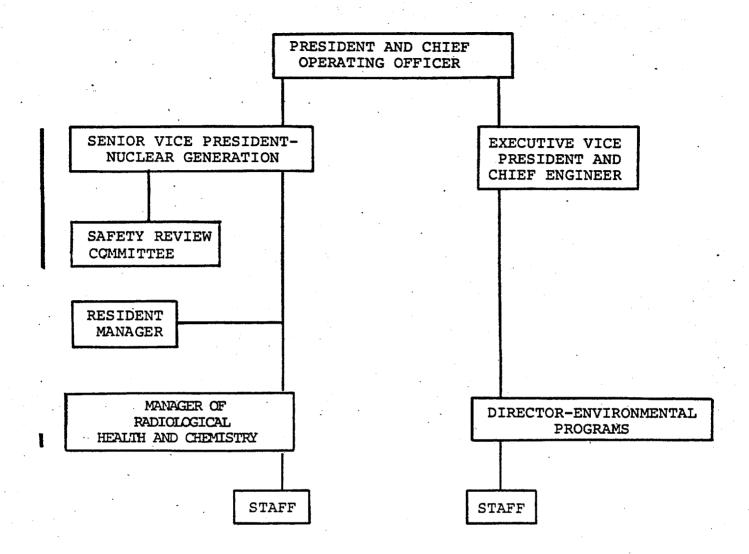


Figure 5.2-2
Management Organization-Offsite
Radiological Environmental
James A. FitzPatrick Nuclear
Power Plant

UNITED STATES NUCLEAR REGULATORY COMMISSION DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 65 to Operating License No. DPR-59 issued to the Power Authority of the State of New York which revises the Technical Specifications for operation of the James A. FitzPatrick Nuclear Plant (the facility) located in Oswego County, New York. The amendment is effective as of the date of its issuance.

The amendment revises the Technical Specifications to allow the reorganization of the FitzPatrick Safety Review Committee.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared—in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated January 6, 1982, (2) Amendment No. 65 to License No. DPR-59, and (3) the Commission's letter to the licensee dated February 3, 1982 . All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 3rd day of February 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief Operating Reactors Branch #2 Division of Licensing