

Docket File

Docket No. 50-333

JUL 2 1981

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Mr. George T. Berry
President and Chief Operating
Officer
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019



Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application dated June 24, 1981.

The enclosed revised FitzPatrick Technical Specifications establish one set of requirements for the three systems which are used to identify primary coolant system leakage. In consonance with the Standard Technical Specifications for Boiling Water Reactors, two of the three systems must be operational during plant power operations. This specification replaces the existing specifications which individually address the systems which monitor primary coolant system leakage.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

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Enclosures:

- 1. Amendment No. 56 to DPR-59
- 2. Safety Evaluation
- 3. Notice

cc w/ enclosures
See next page

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Amend + FIC Notice ONLY
OELD
RGoddard
7/2/81

ORP
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DPickett
7/1/81

J. Olshinski
ORAB
JOlshinski
7/1/81

SNorris *PPolk*

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DATE	7/1/81	7/1/81	7/1/81	7/1/81		

Mr. George T. Berry
Power Authority of the State
of New York

cc:

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Resident Inspector
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated June 24, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-59 is hereby amended to read as follows:

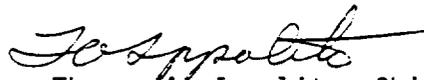
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(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 2, 1981

ATTACHMENT TO LICENSE AMENDMENT NO.56
FACILITY OPERATING LICENSE NO. DPR-59
DOCKET NO. 50-333

Revise Appendix A Technical Specifications as follows:

Delete

141
142
142a

Replace

141
142
142a

4. Except as specified in 3.6.C.3 above, the reactor coolant water shall not exceed the following limits with steaming rates greater than or equal to 100,000 lb/hr and during reactor shutdowns.

Conductivity 5 μ mho/cm
Chloride ion 0.5 ppm

5. If Specification 3.6.C cannot be met, the reactor shall be placed in a cold condition within 24 hr.

D. Coolant Leakage

1. Any time irradiated fuel is in the reactor vessel and reactor coolant temperature is above 212°F, reactor coolant leakage into the primary containment from unidentified sources shall not exceed 5 gpm. In addition, the total reactor coolant system leakage into the primary containment shall not exceed 25 gpm.
2. The following reactor coolant system leakage detection systems shall be operable during reactor power operation:
 - a. Drywell sump monitoring system (equipment drain sump monitoring or floor drain sump monitoring),
 - b. Drywell Continuous Atmosphere (particulate) Radioactivity Monitoring system, and
 - c. Drywell Continuous Atmosphere (gaseous) Radioactivity Monitoring System.

D. Coolant Leakage

Reactor coolant leakage rate inside the primary containment shall be established once/day utilizing the Equipment and Floor Drain Sump Monitoring Systems.

3. With only two of the leakage detection systems operable (3.6.D.2), operation may continue for up to 30 days provided grab samples of the drywell atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous or particulate monitoring system is inoperable; otherwise, be in at least hot shutdown within the next 12 hours and in cold shutdown within the next 24 hours.

3. Drywell Continuous Atmosphere Radioactivity Monitoring System instrumentation shall be functionally tested and calibrated as specified in Table 4.6-2.

E. Safety and Safety/Relief Valves

1. During reactor power operating conditions and prior to startup from a cold condition, or whenever reactor coolant pressure is greater than atmosphere and temperature greater than 212°F,
the safety mode of all safety/relief valves shall be operable, except as specified by Specification 3.6.E.2. The Automatic Depressurization System valves shall be operable as required by Specification 3.5.D.

E. Safety and Safety/Relief Valves

1. At least one half of all safety/relief valves shall be bench checked or replaced with bench checked valves once each operating cycle. The safety/relief valve settings shall be set as required in Specification 2.2.B. All valves shall be tested every two operating cycles.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-59
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

Introduction

Regulatory Guide 1.45, "Reactor Coolant Pressure Boundary Leakage Detection Systems," recommends that there be at least three separate methods to identify reactor coolant system leakage inside containment. Two of these methods should be: (1) sump level and flow monitoring and (2) airborne particulate radioactivity monitors. The third method may be either: (a) primary containment air cooler condensate flow rate monitors or (b) airborne gaseous radioactivity monitors.

The NRC's Standard Technical Specifications (STS) (Reference 1) requires the same recommendations found in the Regulatory Guide. With one of the leakage detection systems inoperable, plant operation may continue for up to 30 days. If the particulate or gaseous monitoring system is inoperable, grab samples must be obtained and analyzed at least once every 24 hours.

The James A. FitzPatrick plant has redundant drywell continuous atmosphere monitoring systems. Each system utilizes a three-channel monitor to provide information on particulate, iodine and noble gas activities in the drywell atmosphere. During normal operation one of the systems is set to monitor particulate while the other monitors gaseous radioactive releases. In addition, there is an equipment drain sump monitoring system and a floor drain sump monitoring system that can identify reactor coolant system leakage.

Although the FitzPatrick plant has the required diversity in leakage detection systems, the plant's technical specifications (Reference 2) have less conservative limiting conditions of operation compared to those found in the NRC's STS's. Action statements included in the FitzPatrick technical specifications include:

- (a) With either sump monitoring system inoperable, reactor operation must terminate after seven days,
- (b) With a redundant component of either sump monitoring system inoperable, reactor operation must terminate after 30 days,
- (c) One of the two drywell continuous monitoring systems may be inoperable without any time restrictions, and

- (d) With both of the drywell continuous monitoring systems inoperable, reactor operation may continue indefinitely provided (1) both of the sump monitoring systems are operable and (2) grab samples of the drywell atmosphere are obtained and analyzed every 96 hours.

By letter dated April 3, 1981 (Reference 3), the licensee proposed amending the FitzPatrick Technical Specifications by deleting item b above. The staff, however, recommended that the entire section regarding reactor coolant leakage detection systems be replaced by Reference 1.

Evaluation

By letter dated June 24, 1981 (Reference 4) the Power Authority superseded the April 3, 1981 proposed Technical Specifications. In essence, the licensee agreed to incorporate the NRC's Standard Technical Specifications regarding the reactor coolant leakage detection system. These specifications comply with the recommendations made in Regulatory Guide 1.45 and are acceptable to the staff. As discussed above, the STS's are more conservative than those presently enforced at FitzPatrick (particularly with regard to operation of the continuous drywell monitoring systems).

Summary

The licensee has proposed to replace that portion of the FitzPatrick Technical Specifications regarding the reactor coolant leakage detection system with Reference 1. The staff has reviewed the proposed change and concludes that the change enhances public health and safety. Therefore the proposed Technical Specification changes as specified in Reference 4 are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 2, 1981

References

1. NRC Standard Technical Specifications for General Electric Plants, Section 3/4.4.3.
2. James A. FitzPatrick Nuclear Power Plant Technical Specifications, Sections 4.6.D and 3.6.D.
3. Letter from Power Authority of the State of New York to NRC (J. P. Bayne to Director, NRR) dated April 3, 1981.
4. Letter from the Power Authority of the State of New York to NRC (J. P. Bayne to T. A. Ippolito) dated June 29, 1981.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-333POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 56 to Operating License No. DPR-59, issued to the Power Authority of the State of New York, which revised the Technical Specifications for operation of the James A. FitzPatrick Nuclear Plant (the facility) located in Oswego County, New York. The amendment is effective as of the date of its issuance.

The amendment revises the Technical Specifications to establish one set of requirements for the three systems which are used to identify primary coolant system leakage.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

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For further details with respect to this action, see (1) the application for amendment dated June 24, 1981, (2) Amendment No. 56 to License No. DPR-59, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C., and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 2nd day of July 1981

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Docket No. 50-333

JUL 2 1981

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Mr. George T. Berry
 President and Chief Operating
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ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief
 Operating Reactors Branch #2
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OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:OR		
SURNAME	SNorris	PPolk:ms	TAIppolito	TMNovak		
DATE	7/1/81	7/1/81	7/1/81	7/1/81		