

October 29, 1982

Docket No. 50-333

Mr. Leroy W. Sinclair
President and Chief Operating Officer
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Sinclair:

The Commission has issued the enclosed Amendment No. 71 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your request dated June 26, 1979.

The amendment revises the Technical Specifications pertaining to the emergency service water system to include an allowance for normal pump wear in the emergency service water pump surveillance test acceptance criteria.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

*Original signed by
D. B. Vassallo*

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 71 to DPR-59
2. Safety Evaluation
3. Notice

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 71
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York (the licensee) dated June 26, 1979 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 71, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 29, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 71

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Remove

p. 240

Insert

p. 240

3.11 (cont'd)

JAFNPP

4.11 (cont'd)

D. Emergency Service Water System

1. To ensure the adequate equipment and area cooling, operable when the requirements of specification 3.5.A and 3.5.B must be satisfied, except as specified below in specification 3.11.D.2.

D. Emergency Service Water System

1. Surveillance of the ESW system shall be performed as follows:

<u>Item</u>	<u>Frequency</u>
a. Simulated Automatic Actuation Test	Each operating cycle
b. Flow Rate Test - ESW pumps shall deliver at least 3,250 gpm against a system head corresponding to a total pump head of ≥ 80 psi, as determined from the pump certification curve by measuring the pump shutoff head which shall be ≥ 117 psi	Once/ 3 months
c. Pump Operability	Once/month
d. Motor Operated Valves	Once/month



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 71 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

I. Introduction

By letter dated June 26, 1979, the Power Authority of the State of New York (licensee) proposed an amendment to the James A. FitzPatrick Nuclear Power Plant (JAFNPP) Technical Specifications (TS) pertaining to the emergency service water system. The change would modify the emergency service water pump surveillance test acceptance criteria to include an allowance for normal pump wear.

II. Background

The licensee's request for amendment was forwarded on June 26, 1979. Shortly thereafter, the NRC's Office of Inspection and Enforcement issued, by letter dated July 11, 1979, IE Bulletin 79-15, Deep Draft Pump Deficiencies. Since the licensee's emergency service water pumps were identified as similar in design to the vertical turbine pump design discussed in the bulletin, we informed the licensee by letter dated July 21, 1979 that approval of the proposed amendment was contingent upon its satisfactory response to the bulletin. The licensee was advised that its application for amendment would be held in abeyance pending receipt of its submittal and completion of our review.

The licensee responded to the bulletin by letters dated August 3 and 9, 1979. Our review of the licensee's response was subsequently conducted and documented in IE Inspection Report No. 50-333/80-15, dated December 31, 1980. In the report, the inspector stated that he had reviewed the documentation pertaining to the bulletin to verify that the licensee's response was timely, accurate, and adequate. No inadequacies were identified by the inspector and the review was considered complete. Thus, action could then proceed on our review of the licensee's proposed amendment.

III. Evaluation

The licensee has proposed to modify TS 4.11.D.1.b, Emergency Service Water System, to include an allowance for normal pump wear in the emergency service water pump surveillance test acceptance criteria.

The flow requirements for the emergency service water pumps are given in

the JAFNPP Final Safety Analysis Report (FSAR) Table 9.7-1, Equipment Supplied by the Emergency Service Water System. A total flow of 2915 gpm is required to meet all equipment needs. The current TS requires that each pump deliver at least 3700 gpm. This flow rate was derived based on a system head corresponding to a total pump head of 80psi and is required to be demonstrated by a pump shutoff head of 120psi. These parameters were selected from the pump head-capacity design curves and reflect new pump performance without regard for wear.

In discussions between the licensee and the pump vendor, an allowance for normal wear wherein pump overhaul should reasonably be considered was discussed and identified. This allowance was four percent and would equate to a revised pump shutoff head of 117psi, rather than the current TS criterion of 120psi. Applying this same four percent allowance for normal wear to the entire design head-capacity pump curve would result in a minimum acceptable pump flow rate of 3250 gpm. This flow rate is 335 gpm above the required pump flow rate of 2915 gpm. Thus, substantial pump capacity in excess of that required for the pumps to perform their intended safety function would exist and still provide allowance for normal pump wear.

We have reviewed the licensee's analyses submitted in support of the proposed amendment dated June 26, 1979 pertaining to the emergency service water system surveillance test acceptance criteria. We have also examined Section 9.7.1, Emergency Service Water System, of the JAFNPP Final Safety Analysis Report Update and Section 6.4, Other Engineered Safety Features, of our corresponding Safety Evaluation of the James A. FitzPatrick Nuclear Power Plant, issued November 20, 1972. Based on that review, we agree with the licensee's analyses and find that the inclusion of an allowance for normal wear in the emergency service water pump head-capacity curves still provides adequate assurance that pump flow rates will be substantially in excess of that required to meet the intended safety function while allowing for normal pump wear. We therefore conclude that the licensee's proposed TS amendment is acceptable.

IV. Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

V. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations

and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Joseph Hegner

Date: October 29, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-333POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 71 to Facility Operating License No. DPR-59 issued to the Power Authority of the State of New York (the licensee), which revises the Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant (the facility), located in Oswego County, New York. The amendment is effective as of the date of its issuance.

The amendment revises the Technical Specifications pertaining to the emergency service water system to include an allowance for normal pump wear in the emergency service water pump surveillance test acceptance criteria.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated June 26, 1979, (2) Amendment No. 71 to License No. DPR-59, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 29th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing