

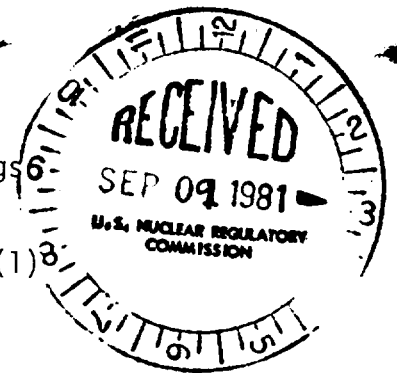
Docket No. 50-333

Mr. George T. Berry
President and Chief Operating
Officer
Power Authority of the State of
New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

Distribution:
✓ Pocket File
NRC PDR
Local PDR
ORB#2 Reading
D. Eisenhut
S. Norris
P. Polk
OELD
OI&E (4)
G. Deegan (4)
B. Scharf (10)
J. Wetmore
ACRS (10)
OPA (C. Miles)

R. Digg
NSIC
TERA
ASLAB (1)



AUG 31 1981

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment is in response to your application dated June 15, 1981.

The amendment revises Section 6.0 Administrative Controls of Appendix A and organization figures of Appendix B to reflect recent organization and management title changes.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

The amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

B10914002B B10831
PDR ADDCK 05000333
P PDR

Enclosures:

- 1. Amendment No. 60
- 2. Notice

audit of notice only

OFFICE		ORB#2:DL	ORB#2:DL	ORB#2:DL	AP:OR:DL	OELD
SURNAME	cc w/encls:	SNorris	P. Polk	T. Ippolito	T. Nbvak	<i>[Signature]</i>
DATE	See next page	8/3/81	8/3/81	8/3/81	8/7/81	8-13-81

Mr. George T. Berry
Power Authority of the State
of New York

cc:

Mr. Charles M. Pratt
Assistant General Counsel
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

U. S. Environmental Protection
Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

Mr. Raymond J. Pasternak
Resident Manager
James A. FitzPatrick Nuclear
Power Plant
P. O. Box 41
Lycoming, New York 13093

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

George M. Wilverding
Manager - Nuclear Licensing
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

State University College at Oswego
Penfield Library - Documents
Oswego, New York 13126

Mr. J. Phillip Bayne
Senior Vice President -
Nuclear Generation
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Resident Inspector
c/o U.S. NRC
P. O. Box 136
Lycoming, New York 13093



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated June 15, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-59 is amended by changes to the Technical Specifications as indicated in the attachment, and Paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 31, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix "A" as follows:

<u>Delete</u>	<u>Insert</u>
247	247
251	251
252	252
252a	252a
253	253
259	259
260	260
141	141

Revise Appendix "B" as follows:

<u>Delete</u>	<u>Insert</u>
43	43
44	44

Administrative Controls are the means by which plant operations are subject to management control. Measures specified in this section provide for the assignment of responsibilities, plant organization, staffing qualifications and related requirements, review and audit mechanisms, procedural controls and reporting requirements. Each of these measures are necessary to ensure safe and efficient facility operation.

6.1 RESPONSIBILITY

The Resident Manager is responsible for safe operation of the plant. During periods when the Resident Manager is unavailable, the Superintendent of Power will assume his responsibilities. In the event both are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel. The Resident Manager reports directly to the Sr. Vice President-Nuclear Generation, as shown in Fig. 6.1-1.

6.2 PLANT STAFF ORGANIZATION

The plant staff organization is shown graphically in Fig. 6.2-1 and functions as follows:

1. A licensed senior reactor operator shall be on site at all times when there is fuel in the reactor.
2. In addition to item 1 above, a licensed reactor operator shall be in the control room at all times when there is fuel in the reactor.
3. In addition to items 1 & 2 above, a licensed reactor operator shall be readily available on site whenever the reactor is in other than cold condition.
4. Two licensed reactor operators shall be in the control room during start-ups and scheduled shutdowns.
5. A licensed senior reactor operator shall be responsible for all movement of new and irradiated fuel within the site boundary. A licensed reactor operator will be required to manipulate or directly supervise the manipulation of the controls of all fuel moving equipment, except the reactor building crane. All fuel movements by the reactor building crane, except new fuel movements from receipt through dry storage, shall be under the direct supervision of a licensed reactor operator. All fuel movements within the core shall be directly monitored by a member of the reactor analyst group. (a)

- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Nuclear Licensing
- m. Emergency Planning
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

- Chairman: Director-Nuclear Design and Analysis
- Vice-Chairman: Vice President-Quality Assurance
- Member: Manager-Nuclear Technical Support
- Member: Manager-Radiological Health and Safety
- Member: Senior Vice President-Nuclear Generation
- Member: Director-Electrical Design and Analysis
- Member: Director of Environmental Programs
- Member: Director-Civil/Structural Design and Analysis
- Member: Director-Mechanical Design and Analysis
- Member: Senior Inservice Inspection Engineer-Nuclear Technical Support
- Member: Manager-Nuclear Licensing
- Member: Senior Nuclear Engineer-Design and Analysis
- Member: Manager-Nuclear Operations

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and six members, including alternates. No more than minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluation for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of code, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDIT

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the President.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. An independent fire protection and loss of prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the President and Chief Operating Officer (to be referred to in the Tech Spec. as the President) on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the President within 14 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the President within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHARTER

6.5.2.11 Conduct of the committee will be in accordance with a charter approved by the President setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

(A) In the event of a Reportable Occurrence, the NRC shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

(B) Each Reportable Occurrence requiring 24 hours notification to the NRC shall be reviewed timely by the PORC and a report submitted by the Resident Manager to the Senior Vice President-Nuclear Generation and the SRC.

6.7 SAFETY LIMIT VIOLATION

(A) If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36 (c) (i).

(B) An immediate report of each safety limit violation shall be made to the NRC by the Resident Manager. The Senior Vice President-Nuclear Generation and Chairman of the SRC will be notified within 24 hours.

(C) The PORC shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluation of: (1) applicable circumstances preceding the occurrence; (2) effects of the occurrence upon facility components or structures and (3) corrective action required to prevent recurrence. The Resident Manager shall forward this report to the Senior Vice President-Nuclear Generation, Chairman of the SRC and the NRC.

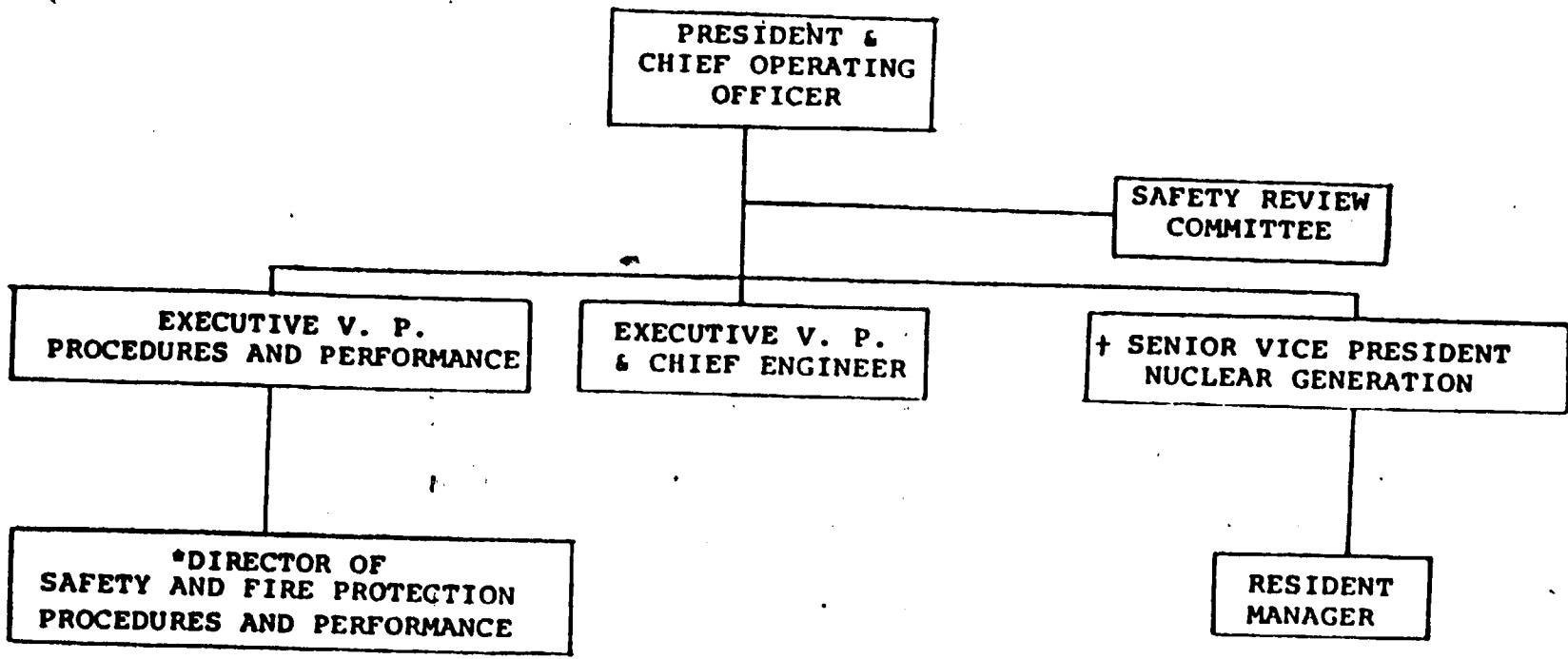
6.8 PROCEDURES

(A) Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5 "Facility Administrative Policies and Procedures" of ANSI 18.7-1972 and Appendix A of Regulatory Guide 1.33, November 1972. In addition, procedures shall be established, implemented and maintained for the Fire Protection Program.

(B) Those procedures affecting nuclear safety shall be reviewed by PORC and approved by the Resident Manager prior to implementation.

(C) Temporary changes to nuclear related procedures may be made provided:

1. The intent of the original procedure is not altered.

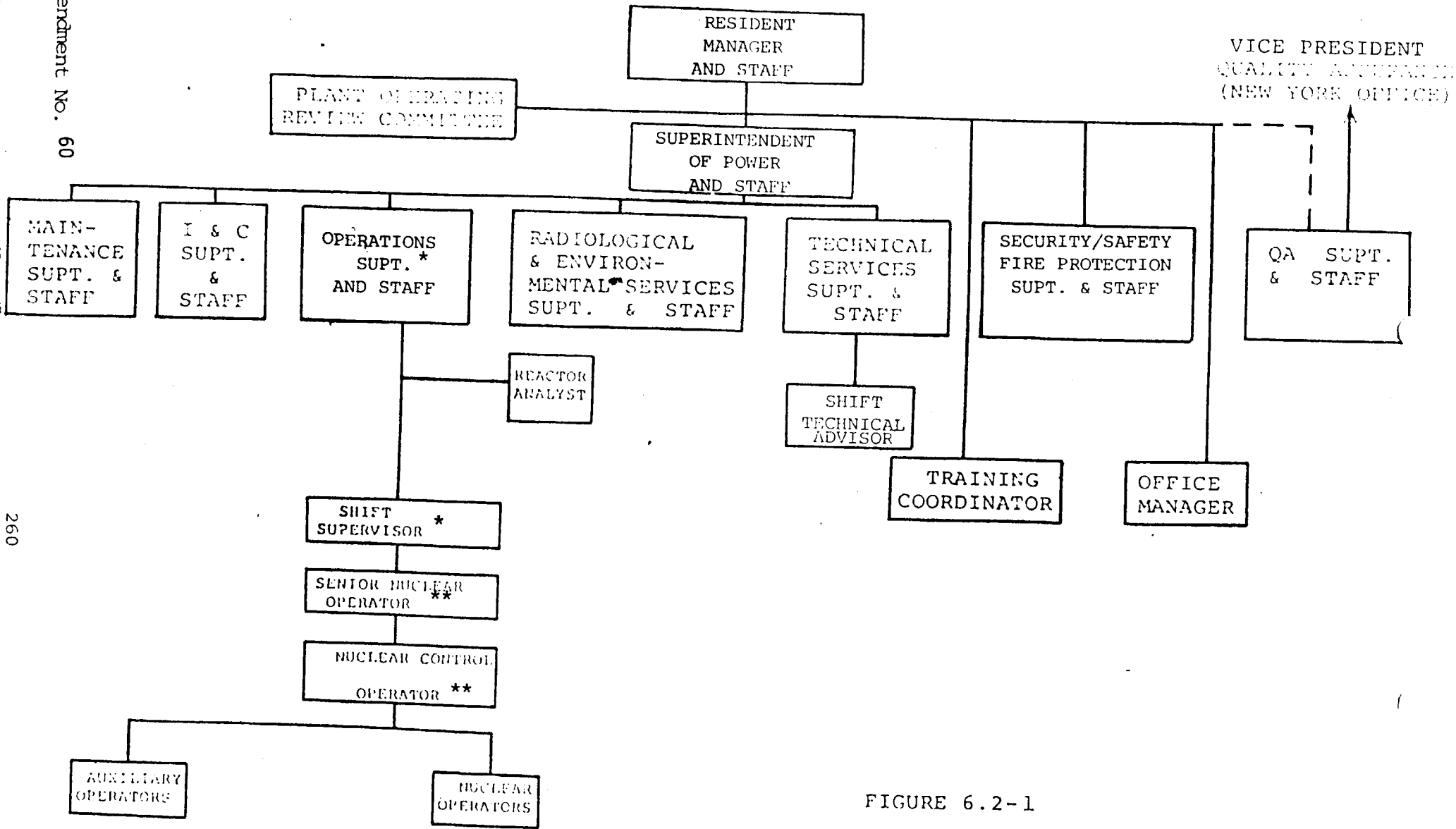


†The Senior Vice President Nuclear Generation is responsible for administering the fire protection program.

*The Director of Safety and Fire Protection is the off-site management position responsible for assessing the effectiveness of the fire protection programs by conducting reviews and audits.

FIGURE 6.1-1

MANAGEMENT ORGANIZATION CHART
JAMES A. FITZPATRICK NUCLEAR POWER PLANT



*SRO - SENIOR REACTOR OPERATOR
 **RO - REACTOR OPERATOR

FIGURE 6.2-1

POWER AUTHORITY OF THE STATE OF NEW YORK
 JAMES A. FITZPATRICK NUCLEAR POWER PLANT
 PLANT STAFF ORGANIZATION

4. Except as specified in 3.6.C.3 above, the reactor coolant water shall not exceed the following limits with steaming rates greater than or equal to 100,000 lb/hr and during reactor shutdowns.

Conductivity 5 μ mho/cm
Chloride ion 0.5 ppm

5. If Specification 3.6.C cannot be met, the reactor shall be placed in a cold condition within 24 hr.

D. Coolant Leakage

1. Any time irradiated fuel is in the reactor vessel and reactor coolant temperature is above 212°F, reactor coolant leakage into the primary containment from unidentified sources shall not exceed 5 gpm. In addition, the total reactor coolant system leakage into the primary containment shall not exceed 25 gpm.
2. The following reactor coolant system leakage detection systems shall be operable during reactor power operation:
 - a. Drywell sump monitoring system (the capability to measure leakage rate from the equipment drain sump monitoring and floor drain sump monitoring),
 - b. Drywell Continuous Atmosphere (particulate) Radioactivity Monitoring system, and
 - c. Drywell Continuous Atmosphere (gaseous) Radioactivity Monitoring System.

D. Coolant Leakage

Reactor coolant leakage rate inside the primary containment shall be established once/day utilizing the Equipment and Floor Drain Sump Monitoring Systems.

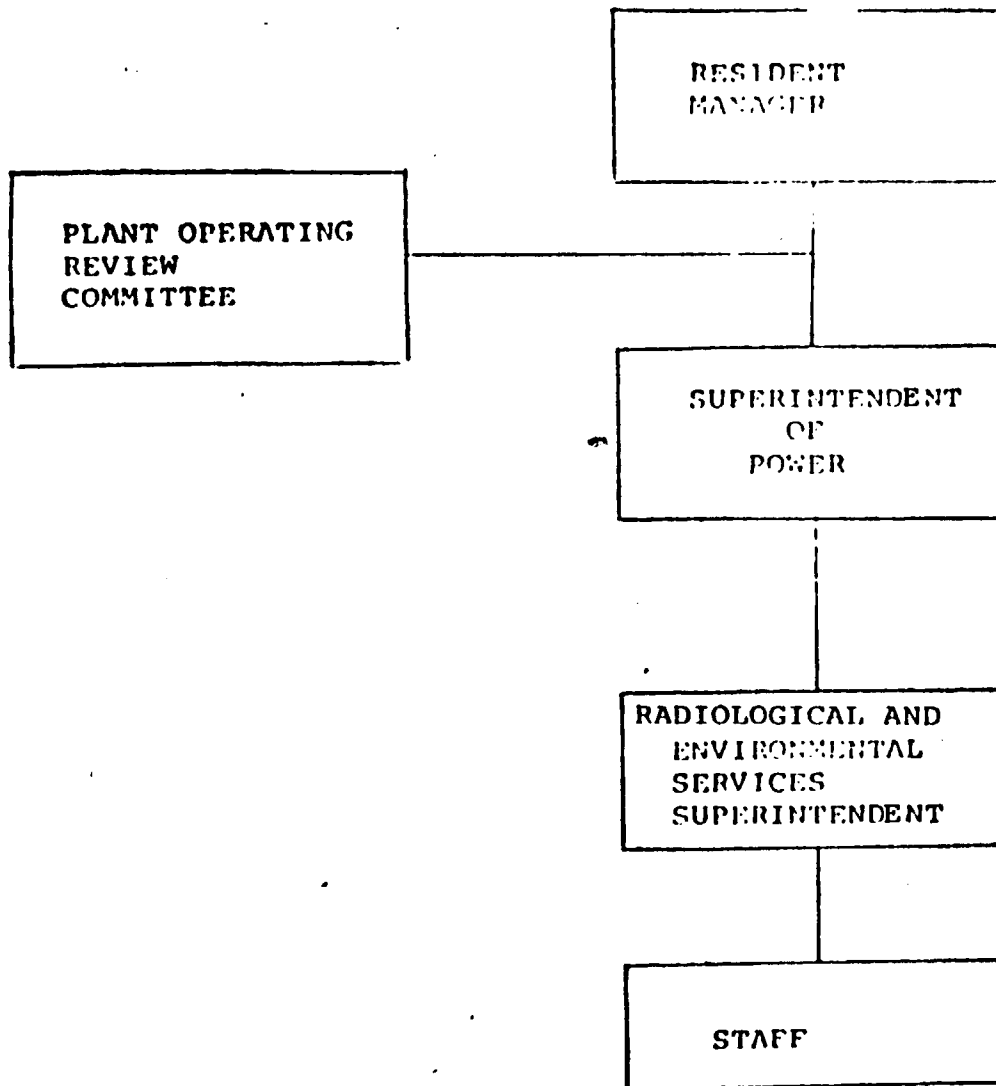


FIGURE 5.2-1
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
PLANT ORGANIZATION - RADIOLOGICAL ENVIRONMENTAL

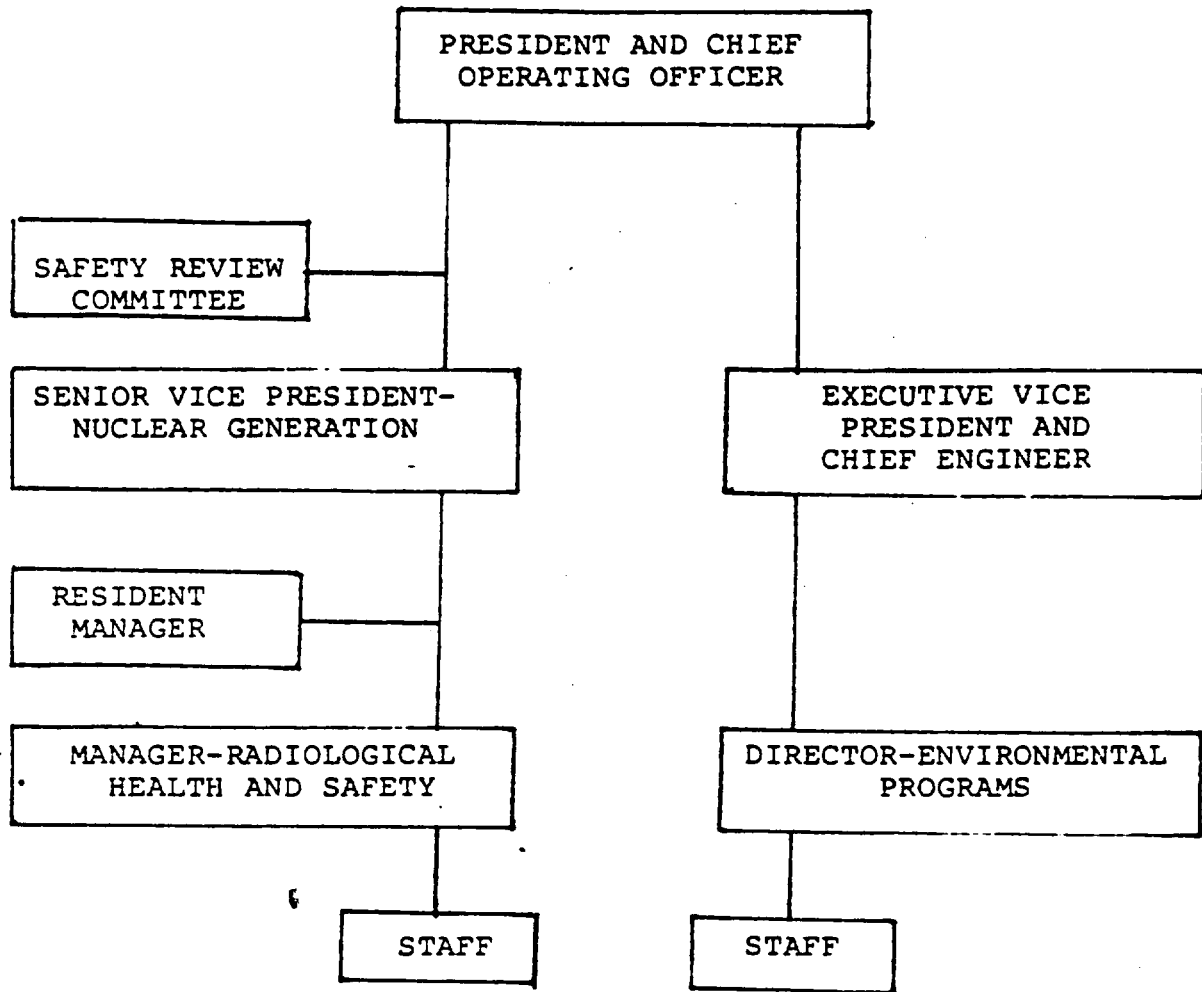


Figure 5.2-2
 Management Organization-Offsite
 Radiological Environmental
 James A. FitzPatrick Nuclear
 Power Plant

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-333POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITYOPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 60 to Facility Operating License No. DPR-59, issued to the Power Authority of the State of New York (the Licensee), which revised the Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant (the facility) located in Oswego County, New York. The amendment is effective as of the date of issuance.

The amendment revises Section 6.0 Administrative Controls of Appendix A and organization figures of Appendix B to reflect recent organization and management title changes.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated June 15, 1981, (2) Amendment No. 60 to License No. DPR-59, and (3) the Commission's letter to the licensee dated August 31, 1981. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A single copy of items (2), and (3), may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 31st day of August 1981

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito
Operating Reactors Branch #2
Division of Licensing