

50-333

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April 13, 1981

Mr. George T. Berry  
President and Chief Operating  
Officer

Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application dated March 26, 1981, regarding interim modification of safety relief valve setpoints.

In your submittal you indicated that a safety relief valve top work had been replaced and that the setpoint for this valve was 1090 psig instead of the Technical Specification limit of 1140 psig. We have reviewed your justification and have concluded that this modification is acceptable for the remainder of the current fuel cycle. Therefore, the enclosed revised Technical Specifications are effective until the end of fuel cycle 5. As agreed to verbally with members of your staff, during your next refueling outage the setpoint for the subject valve will be restored to the original 1140 psig limit.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 54 to License  
No. DPR-59

2. Safety Evaluation

3. Notice

cc w/encs:

See next page

*F.R. Amendment*  
*[Signature]*

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OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:OR	OELD	
SURNAME	SNorris	PPolk:ms	TAIppolito	TMNovak	[Signature]	
DATE	3/30/81	3/31/81	3/31/81	3/31/81	4/3/81	



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 13, 1981

50-333

Mr. George T. Berry  
President and Chief Operating  
Officer  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019


Dear Mr. Berry:

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In your submittal you indicated that a safety relief valve top work had been replaced and that the setpoint for this valve was 1090 psig instead of the Technical Specification limit of 1140 psig. We have reviewed your justification and have concluded that this modification is acceptable for the remainder of the current fuel cycle. Therefore, the enclosed revised Technical Specifications are effective until the end of fuel cycle 5. As agreed to verbally with members of your staff, during your next refueling outage the setpoint for the subject valve will be restored to the original 1140 psig limit.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 54 to License  
No. DPR-59
2. Safety Evaluation
3. Notice

cc w/encls:  
See next page

Mr. George T. Berry  
Power Authority of the State  
of New York

cc:

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Agency  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54  
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated March 26, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 13, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 54  
FACILITY OPERATING LICENSE NO. DPR-59  
DOCKET NO. 50-333

Replace the following pages of the Appendix "A" Technical Specifications  
with the enclosed pages.

Remove

27

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Insert

27

27a

## 1.2 REACTOR COOLANT SYSTEM

### APPLICABILITY:

Applies to limits on reactor coolant system pressure.

### OBJECTIVE:

To establish a limit below which the integrity of the Reactor Coolant System is not threatened due to an overpressure condition.

### SPECIFICATION:

1. The reactor coolant system pressure shall not exceed 1,325 psig at any time when irradiated fuel is present in the reactor vessel.

## 2.2 REACTOR COOLANT SYSTEM

### APPLICABILITY:

Applies to trip settings of the instruments and devices which are provided to prevent the reactor coolant system safety limits from being exceeded.

### OBJECTIVE:

To define the level of the process variables at which automatic protective action is initiated to prevent the safety limits from being exceeded.

### SPECIFICATION:

1. The Limiting Safety System setting shall be specified below:

- A. Reactor coolant high pressure scram shall be  $\leq 1,045$  psig.
- B. Reactor coolant system safety/relief valve nominal settings shall be as follows:

#### Safety/Relief Valves

- 2 valves at 1090 psig
- 2 valves at 1105 psig
- 7 valves at 1140 psig

The allowable setpoint error for each safety/relief valve shall be  $\pm 1$  percent.

## **1.2 REACTOR COOLANT SYSTEM**

### **APPLICABILITY:**

Applies to limits on reactor coolant system pressure.

### **OBJECTIVE:**

To establish a limit below which the integrity of the Reactor Coolant System is not threatened due to an overpressure condition.

### **SPECIFICATION:**

1. The reactor coolant system pressure shall not exceed 1,325 psig at any time when irradiated fuel is present in the reactor vessel.

This page is effective until a cold shutdown of sufficient duration but in any case no later than the next cold shutdown for refueling.

## **2.2 REACTOR COOLANT SYSTEM**

### **APPLICABILITY:**

Applies to trip settings of the instruments and devices which are provided to prevent the reactor coolant system safety limits from being exceeded.

### **OBJECTIVE:**

To define the level of the process variables at which automatic protective action is initiated to prevent the safety limits from being exceeded.

### **SPECIFICATION:**

1. The Limiting Safety System setting shall be specified below:
  - A. Reactor coolant high pressure scram shall be  $\leq 1,045$  psig.
  - B. Reactor coolant system safety/relief valve nominal settings shall be as follows:

#### **Safety/Relief Valves**

3 valves at 1090 psig  
2 valves at 1105 psig  
6 valves at 1140 psig

The allowable setpoint error for each safety/relief valve shall be  $\pm 1$  percent.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-59  
POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
DOCKET NO. 50-333

1.0 Introduction

By letter dated March 26, 1981, the Power Authority of the State of New York (licensee) requested a modification to the James A. FitzPatrick Technical Specifications. This modification involves a change to the setpoints for the reactor coolant system safety/relief valves (SRV). The change has been necessitated by the replacement of the top works of a failed valve. The evaluation of this modification for the remainder of the current FitzPatrick fuel cycle is provided in section 2 below.

2.0 Evaluation

When SRV setpoints are modified three areas of technical concern are affected: (1) spurious opening of SRV's, (2) reactor coolant system peak transient pressure, and (3) structural effects due to SRV discharge. These areas are discussed in turn in the following paragraphs.

By previous amendment (Amendment No. 43) to the FitzPatrick Technical Specifications, SRV's were regrouped with 2 valves set at 1090 psig, 2 valves set at 1105 psig and 7 valves set at 1140 psig. The proposed change will result in one more valve set at 1090 psig, and one less valve set at 1140 psig.

Regarding spurious SRV opening, the addition of a single valve set at 1090 psig will not significantly increase the potential for such action. Since the allowable setpoint error remains at +1% the addition of a low pressure valve has a limited affect when considering inadvertent operation. Therefore, in this regard the proposed modification is considered acceptable.

With respect to reactor coolant peak transient pressure, the effect of the addition of a low pressure relief valve will reduce peak pressure, i.e., an additional SRV will open earlier in the transient. Since the effect is small and in the conservative direction, the modification of the SRV in this regard is considered acceptable.

With respect to structural effects, reducing SRV setpoints will have a less conservative effect on the Torus. More SRV's opening earlier in the transient will result in increased forces exerted against the Torus and its structural support members. Over the past several years Torus modifications have been pursued in accordance with the Mark I containment Long-Term Program. Certain of the modifications (e.g., saddles to reduce torus column loads) have already been accomplished at FitzPatrick. Prior to these modifications the licensee had received permission to continue to operate FitzPatrick with reduced margins of safety in an exemption to GDC 50 issued February 28, 1978. By virtue of the fact that modifications have already been accomplished, the FitzPatrick plant has been restored to margins required for the interim period during which the exemption is in effect. Nevertheless, the FitzPatrick plant will be required to satisfy the Mark I Long Term program requirements, as specified by Order dated January 13, 1981, and, therefore, the change of SRV setpoints in this regard is considered acceptable.

### 3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 13, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-333POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 54 to Operating License No. DPR-59, issued to the Power Authority of the State of New York, which revised the Technical Specifications for operation of the James A. FitzPatrick Nuclear Plant (the facility) located in Oswego County, New York. The amendment is effective as of the date of its issuance.

The amendment changes the Technical Specifications to change the setpoints for the reactor coolant system safety relief valves for the remainder of Cycle 5.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.


The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated March 26, 1981, (2) Amendment No. 54 to License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, April 13, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing