

September 12, 1977

Docket No.: 50-333

Power Authority of the State
of New York
ATTN: Mr. George T. Berry
General Manager and
Chief Engineer
10 Columbus Circle
New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 15, 1977, as supplemented August 25, 1977.

This amendment revises the Inservice Inspection requirements for the plant.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

MZ

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 27
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) sworn to June 13, 1977, as supplemented August 25, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR- 59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 27, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Gerard B. Zvezin for
Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 12, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Replace pages 157, 158, 159, 160, 161, and 162 of the Appendix A Technical Specifications with the attached revised pages. Changes in the revised pages are shown by marginal lines. Pages 157 and 162 are unchanged and are included for convenience only.

TABLE 4.6-1
COMPARISON OF THE JAMES A. FITZPATRICK NUCLEAR POWER PLANT INSERVICE INSPECTION PROGRAM
TO
ASME INSERVICE INSPECTION CODE REQUIREMENTS

Item No.	Examination Category Table IG-251	As Per ASME Code Section XI-IS-200			Extent of Examination, % Intervals		FitzPatrick Proposed Program	
		Components and Parts to be Examined	Examination Method	Extent of Examination, % in 10 Years	10 Years	5 Years	Accessibility	Comments and Examination Methods
1.1	A Pressure containing welds in reactor vessel belt line region	Reactor pressure vessel longitudinal and circumferential welds in core region	Volumetric	Long. S - 10	0	0	The use of doors in the sacrificial shield and thermal insulation provide O.D. access to the required percentage of circumferential and longitudinal welds.	Radiological considerations make it impractical to attempt inspection by any present method. Therefore plans are to inspect these welds only if and when the entire core is removed for other reasons. Ultrasonic methods will then be employed. Inside diameter examination is impractical because sophisticated scanning equipment is not yet available, positioning is extremely difficult, and accurate interpretation of test results is impossible due to the cladding.
				Circ. S - 5 After exposure to 10^{19} NVT of fast neutrons, both long and circ. seams require 50% examination.	0	0		
1.2	B Pressure containing welds in reactor vessel	Closure head circumferential and meridional welds	Volumetric	Circ. S - 5 Merid.S - 10	5 10	0 0	O.D. access to the required percentage of closure head welds and reactor vessel welds above the sacrificial shield are provided by removable insulation panels.	Examination from the outside diameter will be performed utilizing manual or remotely controlled ultrasonic techniques. Although inspection from the I.D. is theoretically possible, it is not presently practical for the same reasons given in Item 1.1 above.
		Reactor pressure vessel longitudinal and circumferential welds above sacrificial shield	Volumetric	Long. S - 10 Circ. S - 5	10 5	0 0		
		Reactor pressure vessel welds within sacrificial shield and above jet pump support plate	Volumetric	Long. S - 10 Circ. S - 5	0 0	0 0	O.D. access to the welds within the sacrificial shield will be provided by the opening of doors in the shield wall and plugs in the insulation.	Radiological considerations preclude periodic inspection. Inspection will be performed only in the event the core must be removed for other reasons. Ultrasonic techniques will then be used. I.D. inspection is impractical for the reasons stated in Item 1.1 above.
		Bottom head circumferential and meridional welds - head to vessel weld and meridional welds outside the vessel support skirt	Volumetric	Circ. S - 5 Merid.S - 10	5 10	0 0	Bottom head welds outside of the support skirt are accessible from the vessel O.D. through use of personnel hatches in the base of the shield wall and removable insulation panels that are provided.	Examination by either manual or remote ultrasonic equipment is possible. Manual techniques will be employed.
		Bottom head circumferential and meridional welds inside support skirt	Volumetric	Circ. S - 5 Merid.S - 10	0 0	0 0	O.D. access inside of the support skirt is provided by access ports in the bottom head insulation and 4 - 18 in. access ports through the support skirt.	Only visual examination will be performed because the large number of penetrations in the area makes volumetric examination impractical.

TABLE 4.6-1 (CONT'D)

Item No.	Examination Category Table 4.6-1	As Per ASME Code Section XI-13-200			FitzPatrick Proposed Program			
		Components and Parts to be Examined	Examination Method	Extent of Examination, % in 10 Years	Extent of Examination, % Intervals		Accessibility	Comments and Examination Methods
					10 Years	1 Year		
1.3	Pressure-containing welds, vessel-to-flange, and head-to-flange	Vessel-to-flange circumferential weld (Vessel flange weld and ligament)	Volumetric	100	100	33	The vessel-to-flange weld is accessible from three locations: The flange mating surface The vessel I.D. The O.D. by means of removable insulation panels.	During refueling, manual ultrasonic examination may be performed from the flange mating surface. If interpretation of the test results is not definitive, inspection will be attempted from the O.D. surface.
		Head-to-flange circumferential weld (Closure head flange and ligament)	Volumetric	100	100	33	The head-to-flange weld is accessible from corresponding surface when the head is removed for refueling.	During refueling, manual ultrasonic examination will be performed, probably from the flange mating surface. If another surface proves more feasible, it will be used.
1.4	Pressure-containing nozzles in reactor vessel	Nozzle-to-vessel attachment welds: Steam nozzle-to-vessel Feedwater nozzle-to-vessel Core spray nozzle-to-vessel Recir. inlet nozzle-to-vessel Recir. outlet nozzle-to-vessel Control rod drive water return nozzle-to-vessel Jet pump instrument nozzle to vessel	Volumetric	100	100	33	Main steam and recirculation outlet nozzle-to-vessel welds are accessible from: The I.D. of the vessel The O.D. by opening doors from sacrificial shield and thermal insulation as required.	Radiation dosage is high. Manual ultrasonic examination from the O.D. will be performed until automated equipment becomes available, but the extent of the examination will be determined by radiological considerations.
		Nozzle-to-closure head attachment welds	Volumetric	100	100	33	All other nozzle welds are accessible from the O.D. by opening doors from the sacrificial shield and thermal insulation as required.	The vessel nozzle inner radii will not be examined from either O.D. or I.D. until automated equipment becomes available due to radiological considerations.
		Inner radius sections of the vessel nozzles listed above	Volumetric	100	0	0	Access to inner radius sections of the various vessel nozzles is subject to the same conditions as stated above for nozzle-to-vessel welds.	The vessel nozzle inner radii will not be examined from either O.D. or I.D. until automated equipment becomes available due to radiological considerations.
		Inner radius sections of the closure head nozzles	Volumetric	100	100	33	Closure head nozzle inner radii become accessible during refueling.	The closure head nozzle inner radii will be examined ultrasonically.
1.5	Pressure-containing welds in vessel penetrations	Welds in vessel at control rod drive penetrations and in-core monitor housings (Stub tube-to-housing and vessel)	Volumetric	0	0	0	Access is provided by observation ports in the bottom head insulation.	Excluded by code from examination.

Page 4-6-1 (cont'd)

Sheet 3 of 6

Item No.	Insulation Category Table 18-231	Components and Parts to be Examined	Insulation Method	Extent of Examination, %	Frequency of Examination, %	Accessibility	Comments and Examination Methods
1.6	Pressure-containing welds in vessel penetrations	Welds in vessel at control rod drive penetrations and in-core monitor (stud tube-to-housing and vessel)	Visual	25	25	Access is provided by observation ports in the bottom head insulation.	Visual examination will be performed using optical equipment capable of providing a complete viewing of the O.D. of the housing external to the vessel for signs of leakage.
1.7	Pressure-containing dissimilar metal welds	Primary nozzles to core-end valves; Rectification inlet; Rectification outlet; Closure nozzles; Core spray	Visual, surface, and volumetric	100	100	Reactor pressure vessel safe-end welds are made accessible by removing thermal insulation and sacrificial shield plugs.	Remote or local visual examination will be performed. The extent to which surface examination is performed is determined by radiological considerations. Manual ultrasonic examination will be performed where possible, with automated equipment, if possible, but the extent of examination will be determined by radiological considerations.
1.8	Pressure-containing dissimilar metal welds	Piping pressure boundary safe-ends in branch piping welds; Piping pressure boundary welds between dissimilar metals	Visual, surface, and volumetric	100	100	Safe ends in branch welds and dissimilar metal welds are made accessible by removing piping thermal insulation.	Remote or local visual examination will be performed. The extent to which surface examination is performed is determined by radiological considerations. Although radiation will be present, manual ultrasonic examination will be performed, where possible, with automated equipment, where the extent of the examination will be determined by radiological considerations.
1.9	Pressure-containing dissimilar metal welds	Fast pressure boundary nozzles-to-safe-end welds	Visual and volumetric	100	0	Not applicable	Not applicable. There are no nozzles to safe-end welds on pumps.
1.9	Pressure-containing dissimilar metal welds	Valve pressure boundary valve to safe-end welds	Visual and volumetric	100	100	Safe ends in valves are made accessible by removing thermal insulation.	Remote or local visual examination will be performed. Although radiation dose is high, manual ultrasonic examination will be performed, where possible, with automated equipment, where possible, but the extent of the examination will be determined by radiological considerations. Exception is taken to volumetric examination of welds which results in drainage of the reactor vessel.

Pilpatrick Proposed Program

As Per AEP Code Section XI-18-230

Extent of Examination, %

Frequency of Examination, %

Comments and Examination Methods

Accessibility

TABLE 3.6-1 (CONT'D)

Item No.	Examination Category Table 3.6-1	Components and Parts to be Examined	Examination Method	Extent of Examination, % in 10 Years	Extent of Examination, % Intervals		Accessibility	Comments and Examination Methods
					10 Years	2 Years		
					Fitzpatrick Proposed Program			
1.8	0-1 Pressure retaining bolting 2 in. and larger diameter	Closure studs and nuts	Volumetric and visual or surface	100	100	33	Closure head stud and nuts are accessible during refueling.	Closure head bolting will be examined under tension when bolting is removed or when the bolted connection is disassembled.
3.3	Pressure retaining bolting 2 in. and larger diameter	Piping pressure boundary pressure-retaining bolting	Visual and volumetric	100	0	0	Not applicable	Not applicable. There is no bolting 2 in. or larger in the piping.
3.4	Pressure retaining bolting 2 in. and larger diameter	Pump pressure boundary pressure-retaining bolting	Visual and volumetric	100	100	33	Reactor recirculation pump casing bolting is accessible during an outage.	Reactor recirculation pump casing bolting will be examined under tension, or when removed or disassembled.
6.4	Pressure retaining bolting 2 in. and larger diameter	Valve pressure boundary pressure-retaining bolting	Visual and volumetric	100	0	0	Not applicable	Not applicable. There is no bolting of this size on valves.
1.9	Pressure retaining bolting 2 in. and larger diameter	Ligaments between threaded stud holes	Volumetric	100	100	33	Ligaments are exposed by removal of the vessel head during refueling.	Examination will be performed using manual ultrasonic techniques.
1.10	Pressure retaining bolting 2 in. and larger diameter	Closure washers and bushings	Visual	100	100	33	Washers and bushings are accessible upon removal of the studs.	Visual examination will be performed only when studs are disassembled.
1.11	0-2 Pressure retaining bolting below 2 in. in diameter	Reactor vessel and closure head	Visual	100	100	33	Accessibility is provided upon removal of insulation.	Visual examination will be made during equipment overhaul.
3.4	Pressure retaining bolting below 2 in. in diameter	Piping pressure boundary pressure-retaining bolting	Visual	100	100	33	" "	Visual examinations of relief valve and BWR system flanges will be made during disassembly for maintenance.
3.3	Pressure retaining bolting below 2 in. in diameter	Pump pressure boundary bolting	Visual	100	0	0	Not Applicable	Not Applicable
6.3	Pressure retaining bolting below 2 in. in diameter	Valve pressure boundary bolting	Visual	100	100	33	Bolting is made accessible by removing thermal insulation.	Visual examination will be made during equipment overhaul.
1.12	II Vessel external supports	Skirt-to-vessel weld	Volumetric	10	10	4	Access is possible from the vessel O.D. by openings in the sacrificial shield wall and removable insulation panels.	Manual ultrasonic examination will be performed from the O.D. of the skirt.

TABLE 4.6-1 (CONT'D)

Item No.	Examination Category (Table 4.6-1)	As Per ASME Code Section III-200		FitzPatrick Proposed Program			Comments and Examination Methods	
		Components and Parts to be Examined	Examination Method	Extent of Examination, % in 10 Years	Extent of Examination, % Intervals			Accessibility
					10 Years	5 Years		
1.14	I-1 Interior clad surface of head	Closure head cladding	Visual and surface or volumetric	100	0	0	Not applicable	Not applicable. The closure head is not clad.
1.14	I-1 Interior clad surface of reactor vessel	Vessel cladding - 6 patches, each 36 sq in. in area	Visual	100	100	50	Vessel cladding is accessible for visual examination above the feed-water sparger.	Visual examination will be made of three patches located in the manually cladded area and three patches located in the automatic cladded area. These areas will be identified for periodic examination.
4.2	J Pressure containing welds in piping	Longitudinal and circumferential welds in piping Residual Heat Removal System Main stem Feedwater Core spray Cleanup Recirculation Control rod drive Closure head High Pressure Coolant Injection System Reactor Core Isolation Cooling System	Visual and volumetric	25	25	8	Accessibility is provided by removable insulation to expose the circumferential welds and, wherever applicable, 15 in. of the longitudinal welds.	Visual examination of all accessible welds will be made, if necessary by remote methods. Volumetric examination, radiographic and/or ultrasonic will be made on water systems - 3.21 in. and larger and on steam systems - 4.29 in. and larger inside diameter, with the following exceptions: 1. Pipe-to-penetration fillet bead circumferential welds because of lack of accessibility. 2. Welds requiring drainage of the reactor vessel.
4.3	K-1 Support members and structures for piping	Piping pressure boundary integrally welded external support attachments	Surface & volumetric	25	25	8	Access will be provided by removable insulation to expose the attachment welds.	Surface examination will be made on fillet welds. Volumetric examination will be made on full penetration welds.
4.6	K-1 Support members and structures for pumps	Pump integrally-welded support attachments	Surface & volumetric	25	25	8	Access will be provided by removable insulation to expose the attachment welds.	Same as 4.5
4.6	K-1 Support members and structures for valves	Integrally-welded support attachments	Surface & volumetric	25	0	0	Not applicable	Not applicable. There are no welded support attachments on valves.
4.6	K-2 Support members and structures for piping	Piping pressure boundary hangers, scrubbers, blowers, and other supports whose structural integrity is relied upon to withstand design loads and seismic induced displacements	Visual	100	100	33	Access of supports is available for visual examination without providing special removable sections.	

TABLE 4.6-1 (CONT'D)

Item No.	Examination Category Table 1B-251	As Per ASME Code Section XI 1B-200			FitPatrick Proposed Program			Comments and Examination Methods
		Components and Parts to be Examined	Examination Method	Extent of Examination, % in 10 Years	Extent of Examination, % Intervals		Accessibility	
					10 Years	5 Years		
5.7	K-2 (Cont'd) Support members and structures for pumps	Pump hangers, snubbers, absorbers, type supports whose structural integrity is relied upon to withstand design loads and seismic induced displacements	Visual	100	100	33	Access of supports is available for visual examination without providing special removable sections.	
6.7	Support members and structures for valves	Valve hangers, snubbers, absorbers, type supports whose structural integrity is relied upon to withstand design loads and seismic induced displacements	Visual	100	100	33	Access of supports is available for visual examination without providing special removable sections.	
5.1	L-1 Pressure containing welds in pump casings	Welds in pump casings	Visual and volumetric	100	0	0	Not applicable	Not applicable. There are no pressure containing welds in pump casings.
5.2	L-2 Pump casings	Internal pressure boundary surfaces of recirculation pumps	Visual	50	50	0	Internal surfaces are accessible for the required visual examination upon disassembly of the pumps. Removable insulation will be provided for pump disassembly.	Visual inspection of the internal surfaces will be made only when recirculation pumps are disassembled for other reasons.
6.1	M-1 Pressure containing welds in valve bodies	Welds in valve bodies on valves 3 in. and larger	Visual and volumetric	100 in each group	0	0	Not applicable	Not applicable. There are no pressure containing seam welds in valve bodies.
6.2	M-2 Surfaces of valve bodies	Internal pressure boundary surfaces of valves 3 in. and larger in main steam, feedwater, reheater, reactor core isolation cooling, core spray, high pressure coolant injection, residual heat removal and recirculation piping system	Visual	One valve of each of these groups of valves of the same construction, design, manufacturing method, manufacturer, and performing similar functions in the system	As required by Code	0	Internal surfaces are accessible for the required visual examination upon disassembly of the valves. Removable insulation will be provided.	Visual examination will be made when valves are disassembled for other reasons. Exception is taken to valves that require draining of the reactor vessel for examination such as those in the recirculation system.
5.5	N Interior surfaces and internal components of reactor vessel	The interior surfaces of the reactor vessel, the internal components, the internal supports welded to the vessel wall, and the space below the reactor core above the bottom head	Visual	Internal welded supports once during each inspection interval, others at the first refueling outage, and then at 3-year intervals	As required by Code	As required by Code	No additional provision is included in the design to provide access to those internals that are not normally accessible from the vessel flange during refueling with the shroud head removed. The lower plenum is accessible by removal of four fuel bundles, control rod, fuel support piece and guide tube.	Visual examination of the lower plenum will be made at the first refueling outage and at subsequent refueling outages at approximately 3-year intervals.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

Introduction

By Application for Amendment to Operating License sworn to June 13, 1977, and supplemented by letter dated August 25, 1977, the Power Authority of the State of New York (the licensee) proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The proposed changes would revise the Inservice Inspection requirements for the plant.

Discussion and Evaluation

Selected welds and heat-affected zones must be inspected periodically to assure continued integrity of the reactor coolant pressure boundary during the service lifetime of the plant. In the application for Operating License, the licensee has stated in the Final Safety Analysis Report (FSAR), Appendix F and in Amendment No. 4 of the FSAR, that the inservice inspection program for the reactor coolant pressure boundary will generally comply with Section XI of the ASME Boiler and Pressure Vessel Code, "Rules for Inservice Inspection of Reactor Coolant System," to the extent practical. Table 4.6-1 of the Technical Specifications provides a schedule for inspection of all affected areas. The licensee proposed changes to Table 4.6-1 which are discussed below:

1. Item 1.4, Category D

"Jet pump instrument nozzle-to-vessel" was proposed to be added to "Nozzle-to-vessel attachment welds" under column "Components and Parts to be Examined." This weld area was originally omitted and since it is a part of the reactor coolant system, it should have been included.

For the line "Nozzle-to-closure head attachment welds" the licensee proposed the following additions:

- a) Under "Examination Method" - add: "Volumetric."
- b) Under "Extent of Examination, % in 10 Years" - add: "100."
- c) Under "Extent of Examination, % Intervals" - add: "100" under the "10 Years" column and "33" under the "5 Years" column.

Since the above proposed additions should have been originally provided, we find them to be acceptable.

2. Item 1.7, Category F

Under "Comments and Examination Methods," the proposed amendment would change "Remote visual examination will be performed" to read "Remote or local visual examination will be performed." This proposed change provides for no change in the end results of the examination; therefore, we find the proposed change acceptable.

3. Item 4.1, Category F

- a) Under "Components and Parts to be Examined," the proposed amendment would add "Piping pressure boundary welds between dissimilar metals." Under "Accessibility," the amendment would change the narrative to read, "Safe ends in branch welds and dissimilar metal welds are made accessible by removing piping thermal insulation."

Although the wording in the 1970 edition of ASME XI makes specific references only to safe end welds under Examination Category F, "Pressure-containing Dissimilar Metal Welds," it was the intent to include all dissimilar metal welds including, but not limited to, safe end welds. We find the proposed change acceptable.

- b) Under "Comments and Examination Methods," the proposed amendment would change "Remote visual examination will be performed" to read "Remote or local visual examination will be performed." This proposed change provides for no change in the end results of the examination; therefore, we find the proposed change acceptable.

- c) Under "Comments and Examination Methods," the proposed amendment would delete "Where practicable radiography will be used in lieu of ultrasonic inspection."

The original inservice inspection baseline was established using ultrasonic testing. Therefore, throughout the entire inservice inspection program, ultrasonic testing will be used where practicable to perform the required volumetric examination. We find the proposed change acceptable.

4. Item 6.3, Category F

Under "Comments and Examination Methods," the proposed amendment would change "Remote visual examination will be performed" to read "Remote or local visual examination will be performed." This proposed change provides for no change in the end results of the examination; therefore, we find the proposed change acceptable.

5. Item 1.8, Category G-1

Under "Comments and Examination Methods," the proposed amendment would change the narrative to read "Closure head bolting will be examined under tension, when the bolting is removed or when the bolted connection is disassembled."

The 1970 edition of ASME XI (Table IS-251, Category G-1) allows bolting 2 inches and larger to be inspected in place under tension, when the bolting is removed or when the bolted connection is disassembled. We find this proposed change acceptable.

6. Item 5.4, Category G-1

- a) Under "Components and Parts to be Examined," the proposed amendment would change "Piping pressure boundary pressure-retaining bolting" to read "Pump pressure boundary pressure-retaining bolting." This would correct a typographical error. We find the proposed change acceptable.
- b) Under "Extent of Examination, % Intervals," the proposed amendment would add "100" under column "10 Years" and "33" under column "5 Years."

Under "Accessibility," the proposed amendment would change "Not Applicable" to read "Reactor Recirculation Pump casing bolting is accessible during an outage."

Under "Comments and Examination Methods," the proposed amendment would change "Not Applicable. There is no bolting of this size on pumps" to read "Reactor recirculation pump casing bolting will be examined under tension, when removed or when disassembled."

The above changes were proposed since it had been determined that the reactor recirculation pump casing studs are greater than 2" in diameter and in accordance with the 1970 edition of ASME XI, should be inspected by Category G-1 requirements. Therefore, we find the above proposed changes acceptable.

7. Item 4.4, Category G-2

Under "Extent of Examination, % Intervals," the proposed amendment would add "100" under column "10 Years" and "33" under column "5 Years." Under "Accessibility," the proposed amendment would delete "Not Applicable" and add "Accessibility is provided upon removal of insulation." Under "Comments and Examination Methods," the proposed amendment would delete "Not Applicable. There is no bolting in piping" and add "Visual examination of Relief Valve and RHR System Flanges will be made during disassembly for maintenance." The above changes were proposed because the piping pressure retaining bolting associated with the Reactor Safety Relief Valves and Residual Heat Removal Spray header flanges was omitted from Table 4.6.1. We find the proposed changes acceptable.

8. Item 5.5, Category G-2

Under "Extent of Examination, % Intervals," the proposed amendment would delete "100" under column "10 Years" and add "0" and delete "33" under column "5 Years" and add "0." Under "Accessibility," the proposed amendment would delete "Bolting is made accessible by removing thermal insulation" and add "Not applicable." Under "Comments and Examination Methods," the proposed amendment would delete "Visual examination will be made during pump overhaul" and add "Not applicable." The above changes were proposed because there is no pump pressure boundary bolting under 2" in diameter within the inservice inspection area defined by the 1970 edition of ASME XI Code. Therefore, we conclude the proposed changes are acceptable.

9. Item 6.5, Category G-2

Under "Examination Method," the proposed amendment would delete "and volumetric." Under "Comments and Examination Methods," the proposed amendment would change the narrative to read "Visual examinations will be made during equipment overhaul." The above changes were proposed since the 1970 edition of the ASME XI Code only requires visual examination to be performed on bolting less than 2". Also, the 1970 addenda to ASME XI deleted the volumetric examination required for valve pressure boundary components. These changes satisfy the intent of the 1970 edition of the ASME XI Code; therefore, we conclude they are acceptable.

10. Item 4.2, Category J

Under "Comments and Examination Methods," the proposed amendment would revise the second sentence to read, "Volumetric examination, radiographic and/or ultrasonic will be made on water systems 2.21" and larger and on steam systems 4.29" and larger inside diameter, with the following exceptions:". The current Technical Specifications require volumetric examination on all sizes of water systems.

Calculations made for the James A. FitzPatrick Plant have determined that postulated failure of water lines smaller than 2.21" nominal pipe size, and steam lines smaller than 4.29" nominal pipe size will not result in loss of reactor coolant that exceeds the capability of normal make-up systems for the interval of time required to permit a reactor shutdown and orderly cooldown from the respective conditions of startup, hot standby, operations, and cooldown. Therefore, in accordance with the provisions of sub-article IS-120 (d) of the 1970 Editions of ASME XI, lines smaller than these nominal pipe diameters are excluded from inservice examinations. We conclude the proposed changes are acceptable.

11. Items 4.5, 5.6, and 6.6, Category K-1

For Items 4.5 and 5.6, under "Examination Method," the proposed amendment would change "Visual and Volumetric" to read "Surface and Volumetric." Under "Comments and Examination Methods," the narrative would be changed to read "Surface examination will be made on all fillet welds. Volumetric examination will be made on all full penetration welds."

For Item 6.6 under "Examination Method," the proposed amendment would change "Visual and Volumetric" to read "Surface and Volumetric."

For the James A. FitzPatrick Plant, integrally welded external support attachment fillet welds to the pressure retaining boundaries of piping, pumps, and valves do not permit volumetric examinations. The difficulty of performing a valid volumetric examination on such weld has been recognized by the ASME Boiler and Pressure Vessel Code Section S.I. Subgroup and the Code now permits (W76) a choice of either volumetric or surface examinations of intergral attachment welds to piping, pumps, and valves. Thus, integrally welded external support attachment fillet welds joining the component supports to the pressure retaining boundaries of piping, pumps, and valves will be given surface examinations. Any full penetration welds joining the component supports to the pressure retaining boundaries of piping, pumps, and valves will be given volumetric examinations. We therefore conclude that the above proposed changes are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 12, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 27 to Facility Operating License No. DPR-59, issued to Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment revises the Inservice Inspection requirements for the plant.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment transmitted by letter dated June 15, 1977, as supplemented August 25, 1977, (2) Amendment No. 27 to License No. DPR-59, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Oswego County Office Building, 46 E. Bridge Street, Oswego, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 12th day of September 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

Gerald B. Zwetzig

Gerald B. Zwetzig, Acting Chief
Operating Reactors Branch #4
Division of Operating Reactors