

Docket No.: 50-333

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MAR. 5 1976

Power Authority of the State of New York
ATTN: Mr. George T. Berry
General Manager and
Chief Engineer
10 Columbus Circle
New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 13 to Facility Operating License No. DPR-59 for the FitzPatrick Nuclear Power Plant.

The amendment consists of a change to the Technical Specifications in response to your application for amendment signed and notarized March 3, 1976. This amendment provides for the replacement of two spring safety valves by two target rock combination safety relief valves in the pressure relief system, a part of the primary coolant system.

Copies of the Safety Evaluation and the Federal Register Notice are enclosed.

Sincerely,

Original signed by

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 13
- 2. Safety Evaluation
- 3. Federal Register Notice

cc w/enclosures: See next page

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DATE	3/5/76	3/5/76	3/5/76	3/5/76	3/5/76	3/5/76

cc w/enclosures:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

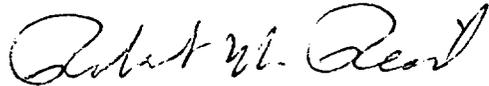
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York and Niagara Mohawk Power Corporation (the licensees) dated March 3, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR. 5 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 13

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Replace pages 27, 28, and 142 of the Appendix A Technical Specifications with the attached pages bearing the same numbers. Changes on these pages are shown by marginal lines.

1.2 REACTOR COOLANT SYSTEM

APPLICABILITY:

Applies to limits on reactor coolant system pressure.

OBJECTIVE:

To establish a limit below which the integrity of the Reactor Coolant System is not threatened due to an overpressure condition.

SPECIFICATION:

1. The reactor coolant system pressure shall not exceed 1.325 psig at any time when irradiated fuel is present in the reactor vessel.

2.2 REACTOR COOLANT SYSTEM

APPLICABILITY:

Applies to trip settings of the instruments and devices which are provided to prevent the reactor coolant system safety limits from being exceeded.

OBJECTIVE:

To define the level of the process variables at which automatic protective action is initiated to prevent the safety limits from being exceeded.

SPECIFICATION:

1. The limiting Safety System setting shall be as specified below:
 - A. Reactor coolant high pressure scram shall be 1.045 psig.
 - B. Reactor coolant system safety/relief valve nominal settings shall be as follows:

Safety/Relief Valves

4 valves at 1.080 psig
4 valves at 1.090 psig
3 valves at 1.100 psig

The allowable setpoint error for each safety/relief valve shall be + 1 percent.

2. The reactor vessel dome pressure shall not exceed 75 psig at any time when operating the Residual Heat Removal pump in the shutdown cooling mode.

2. Action shall be taken to decrease the reactor vessel dome pressure below 75 psig or the shutdown cooling isolation valves shall be closed.

made or found to be inoperable for any reason, continued reactor operation is permissible for the succeeding seven days, unless such subsystem is sooner made operable.

- b. From and after the time that a redundant component of either subsystem is made or found to be inoperable for any reason, continued reactor operation is permissible for the succeeding 30 days, unless component is sooner made operable.

JAFNPP

3. If Specification 3.6.D cannot be met, the reactor shall be placed in a cold condition within 24 hr.

E. Safety and Safety/Relief Valves

1. During reactor power operating conditions and prior to startup from a cold condition, or whenever reactor coolant pressure is greater than atmospheric and temperature greater than 212°F, the safety mode of all safety/relief valves shall be operable, except as specified by Specification 3.6.E.2. The Automatic Depressurization System valves shall be operable as required by Specification 3.5.0.

E. Safety and Safety/Relief Valves

1. At least one half of all safety/relief valves shall be bench checked or replaced with bench checked valves once each operating cycle. The safety/relief valve settings shall be set as required in Specification 2.2.B. All valves shall be tested every two operating cycles.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 13 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

INTRODUCTION

By an application for amendment to Operating License, signed and notarized March 3, 1976, the Power Authority of the State of New York (PASNY) and Niagara Mohawk Power Corporation (NMPC), proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-59, for the James A. FitzPatrick Nuclear Power Plant. The proposed changes provide for the replacement of two spring safety valves by two Target Rock combination safety relief valves in the pressure relief system, a part of the primary coolant system.

EVALUATION

Pressure Relief System

Overpressure protection of the reactor vessel was provided by a combination of two spring-loaded safety valves and nine safety/relief (S/R) combination valves. The nine S/R valves are of dual purpose. The licensee has proposed to replace the two spring-loaded safety valves with two S/R combination valves identical to those nine already in place. The piping runs connected to these two S/R valves and extended through vent pipe to torus suppression pool have been installed previously, in anticipation of this proposed change. These piping runs were designed and fabricated to the same code and quality standards as those associated with the existing nine S/R combination valves.

During the current plant outage, pipes, pipe connections and restraints inside the torus have been inspected and no damage was found. Results of this inspection were confirmed by the Office of Inspection and Enforcement (OI&E), Region I, and are satisfactory.

Based on the results of the inspection on the existing piping runs associated with the existing nine S/R combination valves and the fact that the pressure setpoints of these S/R valves are lower than the two original safety valves (1080 to 1100 psig compared to 1240 psig), we have concluded that the replacement of the spring-loaded safety valves by two S/R dual purpose valves does not degrade the overpressure protection function of the system.

Overpressure Protection

Our review of the existing pressure relief system with nine S/R valves and two safety valves found that the licensee's overpressure analysis was acceptable and provided considerable margin below the allowable limit (1375 psig ASME Code limit). While the addition of two S/R valves would provide still more margin, the licensee does not claim credit for the additional margin. We conclude that the replacement of the two safety valves with two S/R valves is acceptable in regard to ASME Code overpressure protection requirements for the reactor coolant system.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: ~~APR~~ 5 1970

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Facility Operating License No. DPR-59 issued to the Power Authority of the State of New York and the Niagara Mohawk Power Corporation which revised the Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant, located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specifications to provide for the replacement of two spring safety valves by two Target Rock combination safety relief valves in the pressure relief system, a part of the primary coolant system.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

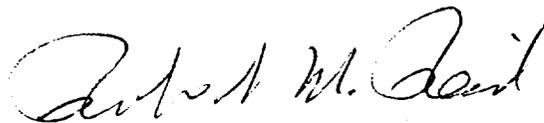
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) application for amendment dated March 3, 1976, (2) Amendment No. 13 to License No. DPR-59, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Oswego City Library, 120 East Second Street, Oswego, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 5th day of March 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors