

Dr. Robert C. Mecredy
 Vice President, Nuclear Operations
 Rochester Gas and Electric Corporation
 89 East Avenue
 Rochester, NY 14649

SUBJECT: ISSUANCE OF AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE
 NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. MA4934)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 74 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated March 1, 1999.

This amendment revises the Ginna Station Improved Technical Specifications battery cell parameters limit for specific gravity (Surveillance Requirement (SR) 3.8.6.3 and SR 3.8.6.6).

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Guy S. Vissing, Senior Project Manager, Section 1
 Project Directorate I
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 74 to License No. DPR-18
 2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 23, 1999

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 74 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

Dr. Robert C. Mecredy
Rochester Gas and Electric Company

R.E. Ginna Nuclear Power Plant

cc:

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Administrator, Monroe County
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Public Service
3 Empire State Plaza, 10th Floor
Albany, NY 12223

DATED: April 23, 1999

AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR
POWER PLANT

Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated March 1, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Section Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.8-19

Insert

3.8-19

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.8.6.1 Verify electrolyte level of each connected battery cell is above the top of the plates and not overflowing.	31 days
SR 3.8.6.2 Verify the float voltage of each connected battery cell is > 2.07 V.	31 days
SR 3.8.6.3 Verify specific gravity of the designated pilot cell in each battery is ≥ 1.195 .	31 days
SR 3.8.6.4 Verify average electrolyte temperature of the designated pilot cell in each battery is $\geq 55^{\circ}\text{F}$.	31 days
SR 3.8.6.5 Verify average electrolyte temperature of every fifth cell of each battery is $\geq 55^{\circ}\text{F}$.	92 days
SR 3.8.6.6 Verify specific gravity of each connected battery cell is: <ul style="list-style-type: none"> a. Not more than 0.020 below average of all connected cells, and b. Average of all connected cells is ≥ 1.195. 	92 days



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated March 1, 1999, the Rochester Gas and Electric Corporation (RG&E or the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise the Ginna Station TS battery cell parameters limit for specific gravity (Surveillance Requirement (SR) 3.8.6.3 and SR 3.8.6.6).

2.0 EVALUATION

The license amendment is requested since the licensee has determined that the battery cell parameters limit for specific gravity that is currently contained within the referenced surveillances is not consistent with the guidance of NUREG-1431, Rev.1, Standard Technical Specifications for Westinghouse Plants. The minimum specific gravity value as recommended by NUREG-1431 is "0.020 below the manufacturer recommended fully charged, nominal specific gravity." Ginna utilizes batteries manufactured by GNB Technologies Industrial Battery Company and the recommended nominal specific gravity is 1.215. Therefore, the minimum value should be 1.195. However, the licensee identified that SR 3.8.6.3 and 3.8.6.6 currently contain minimum limits of 1.188 for A battery and 1.192 for B battery. The values currently in the Ginna TS were based on the actual observed fully charged specific gravity of the batteries at the time of installation, not the manufacturer's nominal value. The immediate corrective actions performed by the licensee as the result of this discovery were to review previous surveillance results to verify operability and to place administrative controls within the Technical Requirements Manual to control the battery cell surveillances in accordance with the more conservative NUREG-1431 guidance. The proposed new minimum value would provide a higher assurance that the battery has sufficient capacity to meet all analyzed conditions and, thus, would provide an increase in safety. Therefore, we have determined that the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (64 FR 14284). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: April 23, 1999