

January 9, 1997

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: ISSUANCE OF AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO.
DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M96959)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 67 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated October 29, 1996.

This amendment revises the MODE of applicability for the motor-driven auxiliary feedwater pump actuation on opening of the main feedwater pump breakers to correct an error introduced during Amendment No. 61.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 67 to
License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

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NAME	GVissing:mw		SLittle	SBajwa	TMarsh	TCollins	
DATE	12/1/96		12/10/96	12/19/96	12/10/96	12/12/96	
OFFICE	TSB	E	OGC				
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DATED: January 9, 1997

AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR POWER
PLANT

Docket File

PUBLIC

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001
January 9, 1997

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Sincerely,

A handwritten signature in cursive script, reading "Guy S. Vissing", is written above the typed name.

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

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Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.67
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated October 29, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.3-20
3.3-27

Insert

3.3-20
3.3-27

3.3 INSTRUMENTATION

3.3.2 Engineered Safety Feature Actuation System (ESFAS) Instrumentation

LCO 3.3.2 The ESFAS instrumentation for each Function in Table 3.3.2-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.2-1.

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each Function.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one channel or train inoperable.	A.1 Enter the Condition referenced in Table 3.3.2-1 for the channel or train.	Immediately
B. As required by Required Action A.1 and referenced by Table 3.3.2-1.	B.1 Restore channel to OPERABLE status.	48 hours
C. Required Action and associated Completion Time of Condition B not met.	C.1 Be in MODE 2.	6 hours

(continued)

Table 3.3.2-1 (page 3 of 3)
Engineered Safety Feature Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
5. Feedwater Isolation						
a. Automatic Actuation Logic and Actuation Relays	1,2(c),3(c)	2 trains	E,G	SR 3.3.2.7	NA	NA
b. SG Water Level - High	1,2(c),3(c)	3 per SG	F,G	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.5	≤ 94%	≤ 85%
c. Safety Injection	Refer to Function 1 (Safety Injection) for all initiation functions and requirements.					
6. Auxiliary Feedwater (AFW)						
a. Manual Initiation						
AFW	1,2,3	1 per pump	N	SR 3.3.2.4	NA	NA
Standby AFW	1,2,3	1 per pump	N	SR 3.3.2.4	NA	NA
b. Automatic Actuation Logic and Actuation Relays	1,2,3	2 trains	E,G	SR 3.3.2.7	NA	NA
c. SG Water Level -Low Low	1,2,3	3 per SG	D,G	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.5	≥ 16%	≥ 17%
d. Safety Injection (Motor driven pumps only)	Refer to Function 1 (Safety Injection) for all initiation functions and requirements.					
e. Undervoltage -Bus 11A and 11B (Turbine driven pump only)	1,2,3	2 per bus	D,G	SR 3.3.2.3 SR 3.3.2.5	≥ 2450 V with ≤ 3.6 sec time delay	≥ 2579 V with ≤ 3.6 sec time delay
f. Trip of Both Main Feedwater Pumps (Motor driven pumps only)	1	2 per MFW pump	B,C	SR 3.3.2.4	NA	NA

(c) Except when all Main Feedwater Regulating and associated bypass valves are closed and de-activated or isolated by a closed manual valve.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated October 29, 1996, Rochester Gas and Electric Corporation (the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise the MODE applicability for the motor-driven auxiliary feedwater (AFW) pump actuation on opening of the main feedwater (MFW) pump breakers to correct an error introduced during Amendment No. 61. Specifically, the mode applicability would be for MODE 1 only and not for MODES 1 and 2 as is currently specified.

2.0 BACKGROUND

On February 13, 1996, the NRC issued Amendment No. 61 to the Ginna Station TSs. This amendment replaced the existing Ginna Station TSs in their entirety with Improved Technical Specifications (ITS) that were based on NUREG-1431. Subsequent to the issuance of the ITS, the licensee discovered an error with respect to Table 3.3.2-1, Function 6.f related to the motor-driven AFW pump actuation on opening of the MFW pump breakers. This error required Function 6.f to be operable in MODES 1 and 2 versus the previous TSs which only required this above 5% power (MODE 1). The addition of MODE 2 for this function has required the licensee to install temporary jumpers to the AFW pump start logic since the MFW pumps are not typically in service under these conditions due to the low system heat loads. Consequently, the purpose of the proposed amendment is to revise the mode applicability for Table 3.3.2-1, Function 6.f to be consistent with TS requirements prior to Amendment No. 61. This change also impacts the Required Actions for limiting condition for operation (LCO) 3.3.2 since in order to exit the MODE applicability for Function 6.f, the plant must only enter MODE 2 and not MODE 3.

3.0 EVALUATION

The MODE applicability for Table 3.3.2-1, Function 6.f is proposed to be revised to remove the requirement for MODE 2. This proposed change is consistent with the Ginna Station TSs prior to Amendment No. 61. The prior TS shows that this function was originally only required above 5% power which is equivalent to the definition of MODE 1 in the ITS.

While standard technical specifications require this function in MODES 1 and 2, the NRC specifically evaluated this difference in the safety evaluation (SE) for issuance of Amendment No. 42, dated May 11, 1981. This SE concluded that only requiring the function above 5% power is appropriate for Ginna Station. This is due to the fact that below 5% power, AFW may be used to maintain steam generator level such that automatic actuation of AFW is not required (nor desired). The addition of MODE 2 in Amendment No. 61 was caused by use of the electronic version of NUREG-1431 in developing the ITS for Ginna Station. Since NUREG-1431 requires this function in Modes 1 and 2, and the licensee failed to identify this difference and make the necessary corrections, Amendment No. 61 was issued with this additional requirement. There have been no subsequent plant modifications or changes to the accident analysis which would invalidate the previous NRC acceptance of only requiring this function above 5% power. Therefore, this change to correct the MODE applicability to reflect requirements particular to the Ginna Station is acceptable.

LCO 3.3.2 Required Action C.1 is proposed to be revised to only require entry into MODE 2 if an inoperable channel of Table 3.3.2-1, Function 6.f is not restored to operable status within 48 hours per Required Action B.1. This change is necessary since the ITS are organized to only require exiting the mode of applicability if a given function is not operable. Since Table 3.3.2-1, Function 6.f is proposed to be revised to only apply in MODE 1, then the necessary required actions should only require exiting MODE 1 (i.e., entering MODE 2) if the function is not operable. There are no other functions in Table 3.3.2-1 which utilize Required Action C.1. The staff has determined this to be acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 64395). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: January 9, 1997