

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

November 10, 1997

SUBJECT: ISSUANCE OF AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M99459)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 69 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated August 19, 1997, as supplemented September 17, 1997.

This amendment revises the Ginna Station Improved Technical Specifications to correct an error in the required accumulator borated water volume specified in Surveillance Requirement 3.5.1.2.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Guy S. Vissing, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 69 to License No. DPR-18  
2. Safety Evaluation

cc w/encls: See next page

*DFOI*

NRC FILE NUMBER 50-244

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PDR ADOCK 05000244  
P PDR

DOCUMENT NAME: G:\GINNA\M99459.AMD



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NAME	GVissing/rs1	SLittle	SBajwa	R. Bachmann	WBeckner	
DATE	10/16/97	10/16/97	10/29/97	10/23/97	10/17/97	

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 10, 1997

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-244

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cc w/encls: See next page

DATED: November 10, 1997

AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR POWER PLANT

Docket File

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Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69  
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated August 19, 1997, as supplemented September 17, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 69 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 10, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.5-2

Insert

3.5-2

**SURVEILLANCE REQUIREMENTS**

SURVEILLANCE	FREQUENCY
SR 3.5.1.1 Verify each accumulator motor operated isolation valve is fully open.	12 hours
SR 3.5.1.2 Verify borated water volume in each accumulator is $\geq$ 1111 cubic feet (50%) and $\leq$ 1139 cubic feet (82%).	12 hours
SR 3.5.1.3 Verify nitrogen cover pressure in each accumulator is $\geq$ 700 psig and $\leq$ 790 psig.	12 hours
SR 3.5.1.4 Verify boron concentration in each accumulator is $\geq$ 2100 ppm and $\leq$ 2600 ppm.	31 days on a STAGGERED TEST BASIS
SR 3.5.1.5 Verify power is removed from each accumulator motor operated isolation valve operator when pressurizer pressure is $>$ 1600 psig.	31 days



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. DPR-18  
ROCHESTER GAS AND ELECTRIC CORPORATION  
R. E. GINNA NUCLEAR POWER PLANT  
DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated August 19, 1997, as supplemented September 17, 1997, the Rochester Gas and Electric Corporation (the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Improved Technical Specifications (ITSs). The requested changes would revise the Ginna Station ITSs to correct an error in the required accumulator borated water volume specified in Surveillance Requirement (SR) 3.5.1.2. The September 17, 1997, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

On February 13, 1996, the NRC issued Amendment No. 61 to the Ginna Technical Specifications (TSs). This amendment replaced the existing TSs in their entirety with the ITSs. Included within the ITS (SR 3.5.1.2) was the addition of specified accumulator borated water volumes to match with the existing specified accumulator percent level values. That is, the previous TSs already required an accumulator inventory based on percent values of instrumentation range. The limits were between 50% and 82% of the instrumentation range. The indicated level of 50% corresponded to 1108 cubic feet of water in the accumulator and the indicated level of 82% corresponded to 1134 cubic feet. However, in SR 3.5.1.2 the indicated level of 50% is currently shown to correspond to 1126 cubic feet and the indicated level of 82% is currently shown to correspond to 1154 cubic feet. In arriving at these volume values, the licensee has subsequently discovered a calculation error in their determination. The correct values, taking into account the correct wall thickness of the accumulator tank, are 1111 cubic feet for the 50% indication and 1139 cubic feet for the 82% indication. The licensee has since re-evaluated the affected accident analyses (e.g., small-and large-break loss-of-coolant accidents and steam line breaks) using the correct cubic feet values with respect to the 50% and 82% levels as specified in the proposed change to SR 3.5.1.2 and determined a negligible change results.

The NRC staff has reviewed the licensee's request and has determined that, since the corrected accumulator water volumes result in negligible change such that all requirements with respect to the emergency core cooling system remain met, the proposed change to SR 3.5.1.2 is acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 52587). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: November 10, 1997