

December 9, 1997

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: ISSUANCE OF AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO.
DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M96529)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 70 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated September 29, 1997, as supplemented October 8, 1997. The September 29 and October 8, 1997 applications, superseded in their entirety the applications dated December 13, 1996, as supplemented April 24, 1997, and June 3, 1997.

This amendment revises the Administrative Controls Section 5.6.6 of the Ginna Technical Specifications to incorporate a reference to the methodology for determining pressure/temperature and low-temperature overpressure protection limits and corrects a typographical error.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 70 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

DOCUMENT NAME: G:\GINNA\M96529.AMD

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	PM:PDI-1	E	LA:PDI-1	D:PDI-1	OGC	NRR:ADPR:TSB
NAME	GVissing/rsf		SLittle	SB	WBeckner	
DATE	11/6/97		11/8/97	11/25/97	11/7/97	

Official Record Copy

acceptance letter he issued
+ cited in TS

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PDR ADDCK 05000244
P PDR



DATED: December 9, 1997

AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-18-GINNA NUCLEAR POWER PLANT

Docket File

PUBLIC

PDI-1 R/F

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OGC

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cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 9, 1997

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Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
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Rochester, NY 14649

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Sincerely,

A handwritten signature in cursive script, reading "Guy S. Vissing", is positioned above the typed name.

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

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2. Safety Evaluation

cc w/encls: See next page

Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. DPR-18

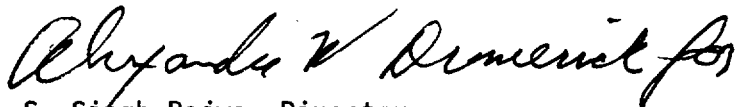
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated September 29, 1997, as supplemented October 8, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 9, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Insert

5.0-22

5.0-22

5.6 Reporting Requirements

5.6.6 PTLR (continued)

- c. The analytical methods used to determine the RCS pressure and temperature and LTOP limits shall be those previously reviewed and approved by the NRC in NRC letter, "R.E. Ginna - Acceptance for Referencing of Pressure Temperature Limits Report, Revision 2 (TAC No. M96529)," dated November 28, 1997. Specifically, the methodology is described in the following documents:
 - 1. Letter from R.C. Mecredy, Rochester Gas and Electric Corporation (RG&E), to Document Control Desk, NRC, Attention: Guy S. Vissing, "Application for Facility Operating License, Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR) Administrative Controls Requirements," Attachment VI, September 29, 1997, as supplemented by letter from R.C. Mecredy, RG&E, to Guy S. Vissing, NRC, "Corrections to Proposed Low Temperature Overpressure Protection System Technical Specification," October 8, 1997.
 - 2. WCAP-14040-NP-A, "Methodology Used to Develop Cold Overpressure Mitigating System Setpoints and RCS Heatup and Cooldown Limit Curves," Sections 1 and 2, January, 1996.
 - d. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for revisions or supplement thereto.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated September 29, 1997, as supplemented October 8, 1997, the Rochester Gas and Electric Corporation (the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise the Administrative Controls Section 5.6.6 of the Ginna TSs to incorporate a reference to the methodology for determining pressure/temperature (P/T) and low-temperature overpressure protection (LTOP) limits and correct a typographical error. The September 29, 1997, and October 8, 1997, submittals superseded in their entirety the requests for amendment dated December 13, 1996, as supplemented April 24, 1997, and June 3, 1997.

2.0 EVALUATION

The proposed amendment follows the guidance given in Generic Letter 96-03 for relocating LTOP and reactor coolant system (RCS) P/T limits to the RCS Pressure and Temperature Limits Report (PTLR). The licensee generated plant-specific P/T and LTOP limits using the NRC-approved WCAP-14040 methodology. The licensee has revised the PTLR curves, setpoints, values and parameters using portions of WCAP-14040, updated data and estimated values for neutron fluence. The staff has reviewed the PTLR and supporting submittals submitted by the licensee and by letter dated November 28, 1997, have found the report acceptable for referencing in the administrative controls section of the TSs in accordance with Generic Letter 96-03 and, therefore, have found the proposed TS change acceptable.

The proposed amendment corrects a minor spelling error in the TSs. We have determined this to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 59921). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: December 9, 1997