

June 4, 1991

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO.
DPR-18 - R. E. GINNA NUCLEAR POWER PLANT (TAC NO. 79831)

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated February 15, 1991.

This amendment revises the Technical Specifications to renumber the radiation monitor from R-20 to R-20A, to add radiation monitor R-20B to Tables 3.5-5 and 4.1-5 as item l.g, and to add a triple asterisk (***) to R-14A to indicate that Table 3.5-6 is also applicable.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

original signed by

Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 43 to License No. DPR-18
- 2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDI-3
MRushbrook
5/16/91

PE:PDI-3
AChu:swsw
5/16/91

PM:PDI-3
AJohnson
5/21/91

SPLB
CMcCracken
5/17/91

OCB
R. Bachmann
5/23/91

PD:PDI-3
RWessman
5/31/91

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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Vice President, Nuclear Production
Rochester Gas & Electric Corporation
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Sincerely,

A handwritten signature in cursive script, appearing to read "Allen Johnson".

Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 43 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

Dr. Robert C. Mecredy

Ginna

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated February 15, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard Wessman, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 4, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
3.5-20	3.5-20
3.5-20a	3.5-20a
4.1-12	4.1-12

Table 3.5-5
Radioactive Effluent Monitoring Instrumentation

		<u>Minimum Channels Operable</u>	<u>Action</u>
1.	Gross Activity Monitors (Liquid)		
a.	Liquid Radwaste (R-18)	1	1
b.	Steam Generator Blowdown (R-19)	1*	2
c.	Turbine Building Floor Drains (R-21)	1	3
d.	High Conductivity Waste (R-22)	1	1
e.	Containment Fan Coolers (R-16)	1	3
f.	Spent Fuel Pool Heat Exchanger A Loop (R-20A)	1+++	3
g.	Spent Fuel Pool Heat Exchanger B Loop (R-20B)	1+++	3
2.	Plant Ventilation		
a.	Without Mini-Purge		
1.	Noble Gas Activity (R-14) (Providing Alarm and Isolation of Gas Decay Tanks)	1	4
2.	Particulate Sampler (R-13)	1	5
3.	Iodine Sampler (R-10B or R-14A***)	1	5
b.	With Mini-Purge		
1.	Noble Gas Activity (R-14)	1	4
2.	Particulate Sampler (R-13)	1	5
3.	Iodine Sampler (R-10B or R-14A***)	1	5
4.	Noble Gas Activity (R-12) or Particulate Sampler (R-11)	1++	8
3.	Shutdown Purge		
a.	Noble Gas Activity (R-12)	1+	8
b.	Particulate Sampler (R-11)	1+	8

		<u>Minimum Channels Operable</u>	<u>Action</u>
	c. Iodine Sampler (R-10A or R-12A***)	1+	5
4.	Air Ejector Monitor (R-15 or R-15A***)	1**	6
5.	Waste Gas System Oxygen Monitor	1	7

- * Not required when Steam Generator Blowdown is being recycled (i.e. not released).
- + Required only during shutdown purges and required to sample the containment stack.
- ++ Required to sample containment during mini-purge operation.
- ** Not required during Cold or Refueling Shutdown.
- *** Also see Table 3.5-6.
- +++ Applicable when Heat Exchanger in service.

3.5-20a

Table 4.1-5

Radioactive Effluent Monitoring Surveillance Requirements

	<u>Instrument</u>	<u>Channel Check</u>	<u>Source Check</u>	<u>Functional Test</u>	<u>Channel Calibration</u>
1.	Gross Activity Monitor (Liquid)				
a.	Liquid Rad Waste (R-18)	D(7)	M(4)	Q(1)	R(5)
b.	Steam Generator Blowdown (R-19)	D(7)	M(4)	Q(1)	R(5)
c.	Turbine Building Floor Drains (R-21)	D(7)	M(4)	Q(1)	R(5)
d.	High Conductivity Waste (R-22)	D(7)	M(4)	Q(1)	R(5)
e.	Containment Fan Coolers (R-16)	D(7)	M(4)	Q(2)	R(5)
f.	Spent Fuel Pool Heat Exchanger A Loop (R-20A)	D(7)	M(4)	Q(2)	R(5)
g.	Spent Fuel Pool Heat Exchanger B Loop (R-20B)	D(7)	M(4)	Q(2)	R(5)
2.	Plant Ventilation				
a.	Noble Gas Activity (R-14) (Alarm and Isolation of Gas Decay Tanks)	D(7)	M	Q(1)	R(5)
b.	Particulate Sampler (R-13)	W(7)	N.A.	N.A.	R(5)
c.	Iodine Sampler (R-10B and R-14A)	W(7)	N.A.	M	R(5)
d.	Flow Rate Determination	N.A.	N.A.	N.A.	R(6)
3.	Containment Purge				
a.	Noble Gas Activity (R-12)	D(7)	PR	Q(1)	R(5)
b.	Particulate Sampler (R-11)	W(7)	N.A.	Q(1)	R(5)
c.	Iodine Sampler (R-10A and R-12A)	W(7)	N.A.	M	R(5)
d.	Flow Rate Determination	N.A.	N.A.	N.A.	R(6)
4.	Air Ejector Monitor (R-15 and R-15A)	D(7)	M	M(2)	R(5)
5.	Waste Gas System Oxygen Monitor	D	N.A.	N.A.	Q(1)
6.	Main Steam Lines (R-31 and R-32)	M	N.A.	Q	R



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated February 15, 1991, Rochester Gas and Electric Corporation (RG&E) (the licensee) requested an amendment to Appendix A of Facility Operating License No. DPR-18 to revise Technical Specification Tables 3.5-5 and 4.1-5. The proposed amendment would renumber the radiation monitor from R-20 to R-20A and add radiation monitor R-20B to Tables 3.5-5 and 4.1-5 as item 1.g. Radiation Monitor R-20B has been installed and in service for several years, but it was not listed specifically in R. E. Ginna's Technical Specifications. In addition, a triple asterisk (***) has also been added to R-14A to indicate that Table 3.5-6 is also applicable.

2.0 EVALUATION

The current R. E. Ginna Technical Specification lists the radiation monitor in the service water line discharge from spent fuel pool heat exchanger "A" as R-20. A second radiation monitor, R-20B, was installed in the service water line discharge from the backup fuel pool heat exchanger "B" which was installed at the same time. By letter dated November 3, 1981, the staff reviewed and approved a proposed spent fuel pool cooling system modification for R. E. Ginna. This modification was implemented in 1986, and consisted of the addition of a new cooling loop in parallel with the existing loop which is sized to accommodate the maximum normal and abnormal heat loads. Radiation monitor R-20B, and the spent fuel pool heat exchanger "B" were installed at about the same time, but R-20B has not been listed in R. E. Ginna's Technical Specifications. Therefore, the licensee proposes to renumber R-20 as R-20A in Tables 3.5-5 and 4.1-5, and add R-20B as item 1.g in these two Tables. In addition, the licensee also proposes to add a triple asterisk to R-14A to indicate that Table 3.5-6 also applies.

The staff's review finds that the proposed Technical Specification changes of renumbering the radiation monitor from R-20 to R-20A, adding radiation monitor R-20B to Tables 3.5-5 and 4.1-5, and adding a triple asterisk to R-14A are administrative in nature and will not impact the safe operation of Ginna Plant. Therefore, the staff concludes that the proposed TS changes are acceptable.

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3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of New York official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 20044). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Chu

Date: June 4, 1991

AMENDMENT NO. 43 TO DPR-18 - R. E. GINNA NUCLEAR POWER PLANT DATED June 4, 1991

DISTRIBUTION:

Docket File 50-244

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Local PDR

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