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PSEG Nuclear LLC

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

MONTHLY OPERATING REPORT
SALEM GENERATING STATION UNIT 2
DOCKET NO. 50-311

In compliance with Section 6.9, Reporting Requirements for the Salem Unit 2 Technical Specifications, the operating statistics for **January 2001** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **January 2001.**

Sincerely,

D. F. Garchow
Vice President - Operations

RBK
Attachments

C Distribution

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DOCKET NO.: 50-311
UNIT: Salem 2
DATE: 2/7/01
COMPLETED BY: R. Knieriem
TELEPHONE: (856) 339-1782

Reporting Period **January 2001**

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)	**1115**		
Maximum Dependable Capacity (MWe-Net)	**1106**		
	Month	**Year-to-date**	**Cumulative**
No. of hours reactor was critical	**744**	**744**	**104721**
No. of hours generator was on line (service hours)	**744**	**744**	**101346**
Unit reserve shutdown hours	**0**	**0**	**0**
Net Electrical Energy (MWH)	**804758**	**804758**	**102108696**

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip/Scram
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

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Summary Of Monthly Operating Experience

- Salem Unit 2 began the month of January 2001 operating at full power.
- On January 12, power was reduced to 42% for scheduled Control Valve testing and equipment repairs.
- Salem Unit 2 returned to full power on January 14, and operated at full power for the remainder of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE SALEM GENERATING STATION – UNIT 2

MONTH January 2001

The following items completed during **January 2001** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

Modification 2EC-3535, Package 5, 21 Wide Range (Hot Leg) Temperature Detector (RTD) Low Level Amplifier Replacement

This modification replaced the Westinghouse-Hagan/7100, Class 1E RTD Low Level Amplifier modules of the Reactor Control and Protection System for the T Hot, T Cold, and T AVG loops with NUS Low Level Amplifiers.

Review of this modification under 10CFR50.59 was required because the replacement of the Wide Range (Hot Leg) Temperature Resistance Temperature Detector (RTD) Low Level Amplifiers constituted a change to the facility as described in the UFSAR. The NUS Low Level Amplifiers are equivalent replacements to the Hagan Low Level Amplifiers and there is no change to the function of the associated instrument loops. Therefore, this change would not increase the probability or consequences of an

accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during January 2001.

Procedures Summary of Safety Evaluations

There were no reportable changes in this category implemented during January 2001.

UFSAR Change Notices Summary of Safety Evaluations

Salem UFSAR Change Notice 00-048, Auxiliary Feedwater System Hydraulic Analysis

This change incorporated the results of Revision 2 to the Auxiliary Feedwater System Hydraulic Analysis, S-C-AF-MDC-0445. Revision 2 evaluated lower required minimum Auxiliary Feedwater flows that are used as a basis for new pump In-service Test limits. The results of the analysis demonstrated that the lower minimum flows would be adequate to support normal operation, and all accident scenarios for which the Auxiliary Feedwater System provides mitigation.

Review of this change under 10CFR50.59 was required because the change constitutes a change to the facility as described in the UFSAR. This evaluation verified that the revised In-service Test acceptance criteria for the Auxiliary Feedwater Pumps would be adequate for all accident scenarios for which the Auxiliary Feedwater System provides mitigation. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition no changes to the Technical Specifications were required.

Salem UFSAR Change Notice 00-050, Mode 3 Steam Line Break Analysis

This change documented the results of a vendor analysis regarding the operability of the Main Steam Isolation Valves during Mode 3 operation. The revision to the UFSAR was based upon a Westinghouse evaluation that considered the effect of reduced steam pressure on the ability to close the Main Steam Isolation Valves in response to a

Main Steam Line break occurring in Mode 3. The evaluation demonstrated that the UFSAR accident analyses remain bounding.

Review of this change under 10CFR50.59 was required because the change constitutes a change to the facility as described in the UFSAR. This change provided clarification to the UFSAR safety analysis regarding Main Steam Isolation Valve operability in Mode 3. This evaluation demonstrated that the UFSAR accident analyses remain bounding for Mode 3 Main Steam Line Breaks. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition no changes to the Technical Specifications were required.

Other - Summary of Safety Evaluations

There were no reportable changes in this category implemented during January 2001.