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Docket No. 50-333

Power Authority of the State of
New York
ATTN: Mr. George T. Berry
General Manager and
Chief Engineer
10 Columbus Circle
New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 2 to Facility License No. DPR-59, for the James A. FitzPatrick Nuclear Power Plant. This amendment includes Change No. 2 to the Technical Specifications, and is in response to your request signed and notarized April 24, 1975.

This amendment revises the trip level settings for the 4 KV Emergency Bus Undervoltage Relays in order to be consistent with the new type of relays that have been installed.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original signed by:
Robert A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Enclosures:

1. Amendment No. 2
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures:
See next page

OK per [signature] 5/2

AP 5/13/75

OFFICE →	RL:ORB#1 <i>MBJ</i>	TR: E/C/S	OELD	RL:ORB#1 <i>[signature]</i>	TR F. Schroeder
SURNAME →	MFairtile:lb	TAIPRO		RAPurple	<i>[signature]</i>
DATE →	4/30/75	4/ /75	4/ /75	5/1/75	5/2/75

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cc w/enclosures:
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and General Counsel
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Mr. Z. Chilazi
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New York, New York 10019

J. Bruce MacDonald, Deputy
Commissioner and Counsel
New York State Department of
Commerce and Counsel to the
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Ecology Action
c/o Richard Goldsmith
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College of Law
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Ms. Suzanne Weber
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120 East Second Street
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Mr. Robert P. Jones, Supervisor
Town of Scriba
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Oswego, New York 13126

Mr. Alvin L. Karkau
Chairman, County Legislature
County Office Building
46 East Bridge Street
Oswego, New York 13126

cc w/enclosures & incoming:
Dr. William E. Seymour
Staff Coordinator
New York State Atomic Energy
Council
New York State Department of
Commerce
112 State Street
Albany, New York 12207

Mr. Paul Arbesman
Environmental Protection Agency
26 Federal Plaza
New York, New York 10007

Anthony Z. Roisman, Esquire
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1712 N Street, NW
Washington, D.C. 20036

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 2
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York and Niagara Mohawk Power Corporation (the licensee) dated April 24, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility License No. DPR-59 is hereby amended to read as follows:

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"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 2."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
Robert A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Attachment:
Change No. 2 to Technical
Specifications

Date of Issuance: MAY 2 1975

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ATTACHMENT TO LICENSE AMENDMENT NO. 2
CHANGE NO. 2 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-59
DOCKET NO. 50-333

Revise Appendix A as follows:

Replace page 70 with the attached revised page 70.

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TABLE 3.2-2 (Cont'd)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System(1)	Trip Function	Trip Level Setting	Total Number of Instrument Channels Provided by Design for Both Channels	Remarks
4 (3)	HPCI Steam Line Low Pressure	100 > P > 50 psig (3)	2 Inst. Channels	Close Isolation Valves in HPCI Subsystem
4	HPCI Steam Line Area Temperature	≤ 40° F. (3) above max. ambient	5 Inst. Channels	Close Isolation Valves in HPCI subsystem
1	HPCI Low Pump Suction pressure	≤ 15 in. Hg vac	1 Inst. Channel	Trips HPCI Turbine
1	RCIC Low pump Suction Pressure	≤ 15 in. Hg vac	1 Inst. Channel	Trips RCIC Turbine
1 per 4KV Bus	4 KV Emergency Bus Undervoltage Relay	85 secondary volts + 5%.2.50 sec + 2% time delay		1. Trips all loaded breakers 2. Dead bus start of diesel
1 per 4KV Bus	4 KV Emergency Bus Under Voltage Timer	2.0 sec ± 0.1 sec	2 Inst.	1. Initiates sequential starting of vital loads 2. Initiates diesel breaker close permissive 3. Initiates bus tie breaker trip

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 2 TO LICENSE NO. DPR-59

(CHANGE NO. 2 TO TECHNICAL SPECIFICATIONS)

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A. PITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

Introduction

By affidavit dated April 24, 1975, the Power Authority of the State of New York requested a change to the Technical Specifications appended to Facility Operating License DPR-59 for the James A. PitzPatrick Nuclear Power Plant. The purpose of the request is to review the trip level settings for the 4 KV Emergency Bus Undervoltage Relays in order to be consistent with the new type of relays that have been installed.

Discussion

The previously installed undervoltage relay had evidenced a pitting problem which in turn caused failure of the relay. The licensee replaced the previous design with the present relay in order to eliminate the pitting problem, but in effectuating the change-over the relay trip level settings in Table 3.2-2 of the Technical Specifications need to be amended to accommodate the new design.

Evaluation

The undervoltage relays previously installed appear to be a misapplication. The action of the contacts created a pitting problem upon restoration of the 4 KV bus. The new relay acts essentially instantaneously and prevents arcing, the cause of the pitting. A 2.5 second time delay before initiation of the relay action is included to accommodate momentary interruptions of the normal power supply.



The set point for the new relay is expressed in potential transformer secondary voltage, corresponding to approximately 70 percent of normal bus voltage and includes the 2.5 second delay mentioned above. The set points for the previously installed relay was expressed in bus voltage with delay times ranging from 1.3 to 4.6 seconds, thus for slow voltage drop-offs, this new scheme will decrease the time required to start the Diesel Generators. However, for sharp voltage drops, going to zero volts, an approximate increase of one second will result for starting the generators. The new relay provides only a backup initiation of the Emergency Diesel Generators. The normal starting sequence is initiated on triple low reactor water level or high drywell pressure in the unlikely event of a design basis accident and a simultaneous loss of offsite power. For initiation of the diesels for small break accidents, using the new relay, or for loss of power transients, not associated with a LOCA, the one second delay in starting the diesels is inconsequential.

The new relay: (1) is expected by the licensee to eliminate the arcing and pitting problem leading to failure of the previous relay, (2) does not delay initiation of any ECCS function, (3) does not significantly delay response to any loss of power transient, and (4) cannot contribute to any new accident not already evaluated. The requested Technical Specification change does not involve any relaxation of safety limits, limiting safety system settings nor does it relax any limiting conditions for operation; it merely changes the trip level setting to accommodate the new design.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: **MAY 2 1975**

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 2 to Facility Operating License No. DPR-59 issued to Power Authority of the State of New York and Niagara Mohawk Power Corporation which revised Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant, located in Scriba, Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment revises the trip level settings for the 4 KV Emergency Bus Undervoltage Relays in order to be consistent with new type of relays that have been installed.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application

for amendment dated April 24, 1975, (2) Amendment No. 2 to License No.

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DPR-59, with Change No. 2, and (3) the Commission's related Safety

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Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. and at the Oswego City Library, 120 East Second Street, Oswego, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this MAY 2 1975

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
Robert A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

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