

Docket

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 SKari (w/o Tech Specs)  
 BScharf (15)  
 TCarter  
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 PCollins  
 SVarga

CHebron  
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 TAbernathy  
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~~ARosenthal~~  
~~NHGoodrich~~

Docket No. 50-333

Power Authority of the State of New York  
 ATTN: Mr. George T. Berry  
 General Manager and  
 Chief Engineer  
 10 Columbus Circle  
 New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 4 to Facility License No. DPR-59, for the James A. FitzPatrick Nuclear Power Plant. This amendment includes Change No. 4 to the Technical Specifications, and is in partial response to your letter of August 11, 1975. Requested changes to Appendix B Technical Specifications are being reviewed for a separate amendment to Facility License No. DPR-59.

This amendment eliminates the requirement for surveillance testing of the High Pressure Coolant Injection and Reactor Core Isolation Cooling systems when a supply of steam is unavailable.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Robert W. Reid, Chief  
 Operating Reactors Branch #4  
 Division of Reactor Licensing

Enclosures:

1. Amendment No. 4
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures:  
 See next page

ORB4  
 Ringram  
 12/9/75

ORB4 *MBF*  
 MFairtile:mt  
 11/11/75  
 12

OELD  
 12  
 11/11/75

RWReid ORB4  
 12/19/75

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Power Authority of the State  
of New York

cc w/enclosures:

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Manager Production Plant Engineering  
Niagara Mohawk Corporation  
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Mr. Z. Chilazi  
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Oswego City Library  
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Oswego, New York 13126

Mr. Robert P. Jones, Supervisor  
Town of Scriba  
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Oswego, New York 13126

Mr. Alvin L. Krakau  
Chairman, County Legislature  
County Office Building  
46 East Bridge Street  
Oswego, New York 13126

cc w/enclosures & incoming:

Dr. William E. Seymour  
Staff Coordinator  
New York State Atomic  
Energy Council  
New York State Department  
of Commerce  
112 State Street  
Albany, New York 12207

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4  
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York and Niagara Mohawk Power Corporation (the licensee) dated August 7, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

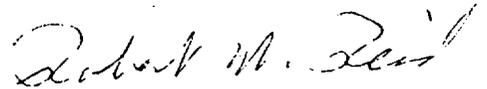
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility License No. DPR-59 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 5."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Reactor Licensing

Attachment:  
Change No. 4 to the  
Technical Specifications

Date of Issuance:  
December 19, 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 4  
CHANGE NO. 4 TO THE TECHNICAL SPECIFICATIONS  
FACILITY OPERATING LICENSE NO. DPR-59  
DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

117 and 118

121 and 122

Insert Pages

117 and 118

121 and 122

4. Should one of the containment cooling subsystems become inoperable, continued reactor operation is permissible for a period not to exceed 7 days, unless such subsystem is sooner made operable provided that during such 7 days all active components of the other containment cooling subsystem, including its associated diesel generator, are operable.
5. If the requirements of 3.5.B cannot be met, the reactor shall be placed in a cold condition within 24 hr.
6. Low power physics testing and reactor operator training shall be permitted with reactor coolant temperature  $\leq 212^{\circ}\text{F}$  with an inoperable component(s) as specified in 3.5.B above.

C. High Pressure Coolant Injection (HPCI) System

1. The HPCI System shall be operable whenever the reactor pressure is greater than 150 psig and irradiated fuel is in the reactor vessel and prior to reactor startup from a cold condition, except as specified below:

C. High Pressure Coolant Injection (HPCI) System

Surveillance of HPCI System shall be performed as follows provided a reactor steam supply is available. If steam is not available at the time the surveillance test is scheduled to be performed, the test shall be performed within ten days of continuous operation from the time steam becomes available.

1. HPCI System testing shall be as specified in 4.5.A.1.a, b, c, d, and f except that the HPCI pump shall deliver at least 4,250 gpm against a system head corresponding to a reactor vessel pressure of 1,120 psig to 150 psig.

- a. From and after the date that the HPCI System is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 7 days unless such system is sooner made operable, provided that during such 7 days all active components of the Automatic Depressurization System, the Core Spray System, LPCI System, and Reactor Core Isolation Cooling System are operable.
  - b. If the requirements of 3.5.C cannot be met, the reactor shall be placed in the cold condition and pressure less than 150 psig within 24 hrs.
2. Low power physics testing and reactor operator training shall be permitted with reactor coolant temperature  $\leq 212^{\circ}\text{F}$  with an inoperable component(s) as specified in 3.5.C.2 above.
- a. When it is determined that the HPCI subsystem is inoperable the RCIC, the LPCI subsystem, both core spray subsystems, and the ADS subsystem actuation logic shall be demonstrated to be operable immediately. The RCIC system and ADS subsystem logic shall be demonstrated to be operable daily thereafter.

E. Reactor Core Isolation Cooling  
(RCIC) System

1. The RCIC System shall be operable whenever there is irradiated fuel in the reactor vessel and the reactor pressure is greater than 150 psig and prior to a reactor startup from a cold condition, except from the time that the RCIC System is made or found to be inoperable for any reason, continued reactor power operation is permissible during the succeeding 7 days unless the system is made operable earlier provided that during these 7 days the HPCI System is operable.
2. If the requirements of 3.5.E cannot be met, the reactor shall be placed in the cold condition and pressure less than 150 psig within 24 hours.
3. Low power physics testing and reactor operator training shall be permitted with inoperable components as specified in 3.5.E. 2 above, provided that reactor coolant temperature is  $\leq 212^{\circ}\text{F}$ .

E. Reactor Core Isolation Cooling  
(RCIC) System

1. RCIC System testing shall be performed as follows provided a reactor steam supply is available. If steam is not available at the time the surveillance test is scheduled to be performed, the test shall be performed within ten days of continuous operation from the time steam becomes available.

<u>Item</u>	<u>Frequency</u>
a. Simulated Automatic Actuation Test	Once/operating cycle
b. Pump Operability	Once/month
c. Motor Operated Valve Operability	Once/month
d. Flow Rate	Once/3 months

The RCIC pump shall deliver a least 400 gpm for a system head corresponding to a reactor pressure of 1,120 psig to 150 psig.

2. When it is determined that the RCIC System is inoperable at a time when it is required to be operable, the HPCI System shall be demonstrated to be operable immediately and daily thereafter.

F. Minimum Emergency Core and Containment Cooling System Availability

1. Any combination of inoperable components in the Core and Containment Cooling Systems shall not defeat the capability of the remaining operable components to fulfill the core and containment cooling functions.
2. When the irradiated fuel is in the reactor vessel and the reactor is in the cold condition, all LPCI, core spray, and containment cooling subsystems may be inoperable provided no work is being done which has the potential for draining the reactor vessel.

G. Maintenance of Filled Discharge Pipe

Whenever core spray subsystems, LPCI subsystems, HPCI, or RCIC are required to be operable, the discharge piping from the pump discharge of these systems to the last block valve shall be filled.

- a. From and after the time that the pump discharge piping of the HPCI, RCIC, LPCI, or Core Spray Systems cannot be maintained in a filled

F. Minimum Emergency Core and Containment Cooling System Availability

Not Applicable.

G. Maintenance of Filled Discharge Pipe

The following surveillance requirements shall be adhered to, in order to assure that the discharge piping of the core spray subsystem, LPCI subsystem, HPCI, and RCIC are filled:

1. Every month prior to the testing of the LPCI subsystem and core spray subsystem, the discharge piping of these systems shall be vented from the high point, and water flow observed.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 4 TO LICENSE NO. DPR-59

(CHANGE NO. 4 TO TECHNICAL SPECIFICATIONS)

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

Introduction

By letter dated August 11, 1975, the licensees requested changes to the Technical Specifications appended to Facility Operating License DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The proposed changes involved both Appendix A and Appendix B Technical Specifications. Only those changes related to Appendix A Technical Specifications are discussed in this report and involve eliminating the requirements for surveillance testing of the High Pressure Coolant Injection (HPCI) and Reactor Core Isolation Cooling (RCIC) systems when a supply of steam is unavailable.

Discussion

The requested change to Technical Specification 4.5, Core and Containment Cooling Systems - Surveillance Requirements, in Sections 4.5.E.1 and 4.5.C.1 was brought about when an Abnormal Occurrence Report filed by the FitzPatrick plant operators pointed out that required periodic surveillance testing could not be performed during extended reactor outages on the High Pressure Coolant Injection (HPCI) or Reactor Core Isolation Cooling (RCIC) systems.

### Evaluation

Both the HPCI and RCIC systems use steam driven turbine pumps. The pumps are required to inject coolant into the reactor vessel, under certain circumstances, in the unlikely event of a loss-of-coolant accident. The reactor, when operating at pressures above 150 psig, is the source of steam for the above-mentioned turbines. When the reactor is shut down for extended outages, scheduled periodic operational tests cannot be performed on the HPCI and RCIC systems as the normal source of required high pressure steam is no longer available. Moreover, these two systems are not required for any safety related function when the reactor system pressure is below 150 psig. Therefore, periodic operational tests on these two systems are not necessary when the reactor system is below 150 psig. No accident not already evaluated can occur as a result of this change. The proposed change does not involve an increase in the probability of an accident, nor contribute to any delay in the injection of coolant in the event of an accident. The requested Technical Specification change does not involve any relaxation of existing safety limits.

### Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the changes does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:  
December 19, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 4 to Facility Operating License No. DPR-59 issued to Power Authority of the State of New York and Niagara Mohawk Power Corporation which revised Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant, located in Scriba, Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment eliminates the requirement for surveillance testing of the High Pressure Coolant Injection and Reactor Core Isolation Cooling systems when a supply of steam is unavailable.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission

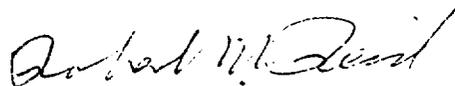
has made appropriate findings as required by the Act and the Commission's rules and regulations in 10. CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated August 7, 1975 (transmitted by letter dated August 11, 1975), (2) Amendment No. 4 to License No. DPR-59, with Change No. 4 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oswego City Library, 120 East Second Street, Oswego, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 19th day of December, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Reactor Licensing