Docket No.: 50-333

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Power Authority of the State of New York
ATTN: Mr. George T. Berry
General Manager and
Chief Engineer
10 Columbus Circle
New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 7 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. This amendment consists of changes to the Technical Specifications in response to your request dated December 2, 1974.

The amendment incorporates into the FitzPatrick Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The technical specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated December 8, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This Guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your technical specifications. Reports that are required by the regulations have not been repeated in your technical specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original Signed by

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing

Enclosures:
1. Amendment No. 7
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/encls: See next page

OFFICE ➤	ORB #4	ORB #4	C-ORB #4			
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Power Authority of the State of New York and Niagara Mohawk Power Corporation (the licensees) dated December 2, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment becomes effective 30 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: JAN — 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 7

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages	Insert Pages
iii and iv	iii and iv
1	1
175 - 176	175 - 176
253 - 254	253 - 254
259 - 271	259 - 267
273	273 - 273a

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TABLE OF CONTENTS (CONT'D)

TECHNICAL SPECIFICATIONS

1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the specifications may be achieved.

A. Reportable Occurrence - A reportable occurrence is defined in Specification 6.12.2, "Reportable Occurrences."

B. Core Alteration - The act of moving any component in the region above the core support plate, below the upper grid and within the shroud. Normal control rod movement with the control rod drive hydraulic system is not defined as a core alteration. Normal movement of in-core instrumentation is not defined as a core alteration.

INTENTIONALLY BLANK

3. Not Applicable

3. Continuous Leak Rate Monitor

f. Radiation Safety

Graduate in engineering or a scientific discipline and 5 years of experience as a technical member or supervisor of the radiation control organization of which at least 2 years have been directly associated with an operating nuclear facility.

g. Safety System Analysis

Graduate in engineering or a scientific discipline with at least 5 years of experience in nuclear engineering, at least 3 years of which have been in technical or technical management positions that perform safety systems analyses of nuclear power plants.

h. Instrumentation and Control

Graduate in engineering or a scientific discipline with at least 5 years of experience in instrumentation and control design, at least 3 years of which are in technical or technical management positions involving nuclear power plant instrumentation and controls.

i. Metallurgy

Graduate in metallurgical engineering or in mechanical engineering with special training in metallurgy, and at least 5 years experience in technical or technical management positions in the metallurgical field, including at least 3 years experience related to nuclear power plants.

6.6 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

A. Any reportable occurrence shall be reported to the Plant Superintendent and General Superintendent Nuclear Generation and reviewed by the Site Operations Review Committee.

B. The Site Operations Review Committee shall prepare a separate report for each reportable occurrence. This report shall include an evaluation of the cause of the occurrence, a record of the corrective action taken, and recommendations for appropriate action to prevent or reduce the probability of a recurrence.

C. Copies of all such reports shall be submitted to the Plant Superintendent, the General Superintendent of Nuclear Generation, and to the Chairman of the Safety Review and Audit Board.

D. The General Superintendent of Nuclear Generation shall report the circumstances of any reportable occurrence to the NRC as specified in Section 6.12 "Plant Reporting Requirements".

6.7 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

A. If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10CFR50.36(c)(1)(i).

B. An immediate report of each safety limit violation shall be made to the Plant Superintendent, General Superintendent of Nuclear Generation, and the Chairman of the Safety Review and Audit Board.

C. The General Superintendent of Nuclear Generation shall immediately make a Reportable Occurrence Report to the AEC as specified in Section 6.12 in the event a safety limit is exceeded.

D. The Site Operation Review Commitee shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluations of: (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility components, systems, or structures, and (3) corrective action required to prevent recurrence. This report shall be submitted to the Plant Superintendent, General Superintendent of Nuclear Generation, the Chairman of the Safety Review and Audit Board, and to the AEC.

6.8 OPERATING PROCEDURES

A. Detailed written procedures, including applicable checkoff lists and instructions, covering areas listed below shall be prepared, approved as specified in Section 6.8.2, and adhered to for operation of all systems and components involving nuclear safety.

1. Normal startup, operation, and shutdown of the reactor and of all systems and components involving nuclear safety of the facility.

2. Refueling operations.

3. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components, including responses to alarms, suspected primary

B. Any actual or attempted introduction into the Generating Station Area of any dangerous weapon, explosive or material capable of producing injury or damage to persons or property, or that in any way could seriously affect the safe operation of the generating units, shall be reported immediately upon detection to the shift supervisor and plant security forces.

C. Investigations shall be conducted by Niagara Mohawk Power Corp. Security Personnel of all attempted or actual security infractions in cooperation with any Federal, State, or local agencies involved, and a report filed with the Plant Superintendent, General Superintendent Nuclear Generation, and Chairman Safety Review and Audit Board.

D. The General Superintendent Nuclear Generation shall report all instances of security violations described in Section 6.10.B as specified in Section 6.12.

6.11 EMERGENCY PLAN

A. A Site Emergency Plan shall be maintained throughout the life of the plant in accordance with the provisions of 10CFR50, Appendix E.

B. Site evacuation exercises shall be conducted annually utilizing applicable provisions contained within the Emergency Plan. This exercise shall involve coordination with offsite support groups and include communication checks.

C. The Emergency Plan and implementing procedures shall be reviewed on an annual basis by the Site Operations Review Committee and Safety Review and Audit Board.

6.12 PLANT REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.12.1. Routine Reports

a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

b. Annual Operating Report.[1/] Routine operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of

previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

(1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 6.12.1.b.(2)(e) below.

(2) For each outage or forced reduction in power[2/] of over twenty percent of design power level where the reduction extends for greater than four hours:

(a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);

(b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;

(c) corrective action taken to reduce the probability of recurrence, if appropriate;

(d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,[3/] use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);

(e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

(f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[4/] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.12.2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 6.12.2.a(5), 6.12.2.a(6), or 6.12.2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 6.12.2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.12.2.a(5) and 6.12.2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.12.2.b(2) and 6.12.2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 6.12.2.b(1) and 6.12.2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 6.12.2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.12.3 Unique Reporting Requirements

Fifteen copies of the Evaluation Report of the results of the first five years of performance of the nondestructive inspections listed in Table 4.6-1 of Technical Specification 4.6.F, Structural Integrity, relating to the FitzPatrick inservice inspection program shall be submitted to the NRC, Director of Reactor Licensing, within three months of the completion of the fifth year of the program.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

2. The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

3. The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

4. This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

TABLE 6.5-1 (CONT'D)

RESPONSIBILITIES AND AUTHORITY OF SAFETY REVIEW ORGANIZATION
JAMES A. FITZPATRICK NUCLEAR STATION

SITE OPERATIONS REVIEW COMMITTEE	SAFETY REVIEW AND AUDIT BOARD	GENERAL MANAGER/ CHIEF ENGINEER
	b. The performance of the entire facility staff relative to nuclear safety. c. The results of all actions taken to correct anomalies occurring in the facility, equipment, structures, systems or method of operation d. The adequacy of the Quality Assurance Program to meet the criteria specified in 10CFR50, Appendix B. e. Any other area of facility operation considered appropriate by the Board or the Vice President-Electrical Operations, and the V.P.-Eng. NMPC	
Investigate all violations of applicable statutes, regulations, orders, license requirements, or internal procedures or instructions having safety significance on Plant operation. Prepare and forward a report covering their evaluation and recommendations to prevent recurrence, to General Supt. Nuclear Generation and the Chairman of Safety Review and Audit Board.	Review violations of applicable statutes, regulations, orders, license requirements, or internal procedures or instructions having safety significance on Plant operation. Submit safety analysis to V.P.-Electric Operations and V.P.-Eng. NMPC	

JAFNPP

TABLE 6.5-1 (CONT'D)

RESPONSIBILITIES AND AUTHORITY OF SAFETY REVIEW ORGANIZATION
JAMES A. FITZPATRICK NUCLEAR STATION

SITE OPERATIONS REVIEW COMMITTEE	SAFETY REVIEW AND AUDIT BOARD	GENERAL MANAGER/ CHIEF ENGINEER
Investigate all violations of the Tech. Specs. (including reportable occurrences) and prepare and forward a report covering their evaluation and recommendations to prevent recurrence to the General Supt. Nuclear Generation and the Chairman of Safety Review and Audit Board.	Review violations of Tech. Specs. (including reportable occurrences). Submit safety analysis to V.P.-Electric Operations and V.P.-Eng. for formal submittal to NRC.	To be advised for concurrence with formal submittal.
Perform special reviews and investigations and render reports thereon as requested by the Chairman of the Safety Review & Audit Board, and the General Superintendent Nuclear Generation.	Review reports and meeting minutes of Site Operations Review Committee.	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 7 TO FACILITY LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

Introduction

By letter dated December 2, 1974, the Power Authority of the State of New York proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-59, for the James A. FitzPatrick Nuclear Power Plant. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and an abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrence" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications," Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence." The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with the licensees and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensees' program for reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications," Revision 4.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: [illegible] 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 7 to Facility Operating License No. DPR-59 issued to Power Authority of the State of New York and Niagara Mohawk Power Corporation which revised Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant, located in Scriba, Oswego County, New York. The amendment becomes effective 30 days after the date of issuance.

This amendment revises the reporting requirements of the Technical Specifications for the James A. FitzPatrick Nuclear Power Plant.

The application for the amendment dated December 2, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 2, 1974, (2) Amendment No. 7 to License No. DPR-59, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Oswego City Library, 120 East Second Street, Oswego, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 7th day of January 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing