August 28, 1991

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. DPR-18 - R. E. GINNA NUCLEAR POWER PLANT (TAC NO. 60035)

The Commission has issued the enclosed Amendment No. 45 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated October 16, 1985, as supplemented on January 14, 1991.

This amendment would revise containment internal pressure limitations to be consistent with the initial conditions assumed in the Westinghouse Electrical Corporation's Boric Acid Storage Tanks (BAST) analysis and Rochester Gas and Electric Corporation's containment integrity analysis.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

original signed by

Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 45 to License No. DPR-18
2. Safety Evaluation

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cc w/enclosures:
See next page

| OFC | :LA:PDI-3 | :PM:PDI-3 | :OGC | :PD:PDI-3(A) | : |
|---|---|---|---|---|---|
| NAME | :MRushbrook | :AJohnson/vsb | : | :SShankman | : |
| DATE | :8/9/91 | :8/13/91 | :8/19/91 | :8/28/91 | : |

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AMENDMENT NO. 45 TO DPR-18 - R. E. GINNA NUCLEAR POWER PLANT DATED

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated October 16, 1985, as supplemented on January 14, 1991 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 45 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective immediately and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Marlon B. Fairtile for

Susan F. Shankman, Acting Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

| Remove | Insert |
|---|---|
| 3.6-1 | 3.6-1 |
| 3.6-3 | 3.6-3 |

## 3.6 Containment System

**Applicability:**

Applies to the integrity of reactor containment.

**Objective:**

To define the operating status of the reactor containment for plant operation.

**Specification:**

### 3.6.1 Containment Integrity

a. Except as allowed by 3.6.3 containment integrity shall not be violated unless the reactor is in the cold shutdown condition.

b. The containment integrity shall not be violated when the reactor vessel head is removed unless the boron concentration is greater than 2000 ppm.

c. Positive reactivity changes shall not be made by rod drive motion or boron dilution whenever the containment integrity is not intact unless the boron concentration is greater than 2000 ppm.

### 3.6.2 Internal Pressure

If the internal pressure exceeds 1 psig or the internal vacuum exceeds 2.0 psig, the condition shall be corrected within 24 hours or the reactor rendered subcritical.

**Basis:**

The reactor coolant system conditions of cold shutdown assure that no steam will be formed and hence there would be no pressure buildup in the containment if the reactor coolant system ruptures.

The shutdown margins are selected based on the type of activities that are being carried out. The (2000 ppm) boron concentration provides shutdown margin which precludes criticality under any circumstances. When the reactor head is not to be removed, a cold shutdown margin of 1%$\Delta$k/k precludes criticality in any occurrence.

Regarding internal pressure limitations, the containment design pressure of 60 psig would not be exceeded if the internal pressure before a major steam break accident were as much as 1 psig.[1] The containment is designed to withstand an internal vacuum of 2.5 psig.[2] The 2.0 psig vacuum is specified as an operating limit to avoid any difficulties with motor cooling.

**References:**

(1) Westinghouse Analysis, "Report for the BAST Concentration Reduction for R. E. Ginna," August 1985.

(2) UFSAR - Section 6.2.1.4.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

## 1.0 INTRODUCTION

By letter dated October 16, 1985, as supplemented on January 14, 1991, the Rochester Gas and Electric Corporation (the licensee) requested an amendment to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. The proposed amendment would change the Ginna Technical Specifications (TS) Section 3.6.2 and its associated Bases to limit the containment internal pressure to 1 psig in lieu of the current 3 psig TS limit. The January 14, 1991, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

## 2.0 EVALUATION

The containment internal pressure limitations are based on a Westinghouse analysis, "Report for the BAST Concentration Reduction for R. E. Ginna," dated August 1985. BAST are the initials for the Boric Acid Storage Tank. This analysis has been approved in our License Amendment No. 33, dated March 30, 1989. We concluded that the containment design pressure limit of 60 psig would not be exceeded if the internal containment pressure prior to a major steam line break accident was limited to 1 psig. The requested decrease in initial internal containment pressure from 3 to 1 psig is in the conservative direction; therefore, it would be more unlikely that a major steam line break accident would cause the containment design pressure to exceed its limit of 60 psig. Based on the above analysis, we conclude that the change in TS Section 3.6.2 is acceptable

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types,

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of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (50FR49792). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Goel

Date: August 28, 1991