

August 9, 1985

Docket No. 50-244  
LS05-85-08-010

Mr. Roger W. Kober, Vice President  
Electric and Steam Production  
Rochester Gas & Electric Corporation  
89 East Avenue  
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: CORRECTION TO SEP RELATED TECHNICAL SPECIFICATIONS (TS)

Re: R. E. Ginna Nuclear Power Plant

By letter dated July 30, 1985, we issued Amendment No. 11 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant which incorporated the subject TS.

The change to TS page 4.4-11 was issued on an incorrect version of this page. Please replace the previously issued page 4.4-11 with the enclosed corrected page.

We regret any inconvenience this error may have caused.

Sincerely,

Original signed by:

John A. Zwolinski, Chief  
Operating Reactors Branch No. 5  
Division of Licensing

Enclosure:  
Corrected TS Page 4.4-11

cc w/enclosure:  
See next page

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Mr. Roger W. Kober  
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc:  
Harry H. Voigt, Esquire  
LeBoeuf, Lamb, Leiby and MacRae  
1333 New Hampshire Avenue, N.W.  
Suite 1100  
Washington, D.C. 20036

Ezra Bialik  
Assistant Attorney General  
Environmental Protection Bureau  
New York State Department of Law  
2 World Trade Center  
New York, New York 10047

Resident Inspector  
R.E. Ginna Plant  
c/o U.S. NRC  
1503 Lake Road  
Ontario, New York 14519

Stanley B. Klimberg, Esquire  
General Counsel  
New York State Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Supervisor of the Town of Ontario  
1850 Ridge Road  
Ontario, New York 14519

Jay Dunkleberger  
Division of Policy Analysis & Planning  
New York State Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

the tendon containing 6 broken wires) shall be inspected. The acceptance criterion then shall be no more than 4 broken wires in any of the additional 4 tendons. If this criterion is not satisfied, all of the tendons shall be inspected and if more than 5% of the total wires are broken, the reactor shall be shut down and depressurized.

#### 4.4.4.2 Pre-Stress Confirmation Test

- a. Lift-off tests shall be performed on the 14 tendons identified in 4.4.4.1a above, at the intervals specified in 4.4.4.1b. If the average stress in the 14 tendons checked is less than 144,000 psi (60% of ultimate stress), all tendons shall be checked for stress and retensioned, if necessary, to a stress of 144,000 psi.
- b. Before reseating a tendon, additional stress (6%) shall be imposed to verify the ability of the tendon to sustain the added stress applied during accident conditions.

#### 4.4.5 Containment Isolation Valves

- 4.4.5.1 Each isolation valve specified in Table 3.6-1 shall be demonstrated to be operable in accordance with the Ginna Station Pump and Valve Test Program submitted in accordance with 10 CFR 50.55a.

#### 4.4.6 Containment Isolation Response

- 4.4.6.1 Each containment isolation instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST operations for the MODES and at the frequencies shown in Table 4.1-1.
- 4.4.6.2 The RESPONSE TIME of the containment isolation valves, as listed in Table 3.6-1, shall be demonstrated to be within the limit at least once per 18 months. This response time includes only the valve travel times for all valves that change position.