

Docket  
file

JAN 30 1976

Docket No. 50-333

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 J R Buchanan

Power Authority of the State of New York  
 ATTN: Mr. George T. Berry  
 General Manager and  
 Chief Engineer  
 10 Columbus Circle  
 New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-59 for the FitzPatrick Nuclear Power Plant.

The amendment consists of a change to the Technical Specifications in response to your application for amendment signed and notarized January 23, 1976. This amendment requires one circuit of the Standby Gas Treatment System to be operable during fuel handling operations and a minimum of one pair of emergency diesel generators to be operable when secondary containment integrity is required.

Copies of the Safety Evaluation and the Federal Register Notice are enclosed.

Sincerely,

Original signed by

Robert W. Reid, Chief  
 Operating Reactors Branch #4  
 Division of Operating Reactors

Enclosures:

1. Amendment No. 10
2. Safety Evaluation
3. Federal Register Notice

cc:

See next page

OFFICE ▶	ORB4	ORB4 <i>MT</i>	OELD <i>Smith</i>	ORB4 <i>Reid</i>		
SURNAME ▶	RIngram	MFairtile:mt	<i>Asp...</i>	RWReid		
DATE ▶	1/29/76	1/29/76	1/30/76	1/30/76		

Power Authority of the State  
of New York

-2-

January 30, 1976

cc w/enclosures:

Scott B. Lilly, General Counsel  
Power Authority of the  
State of New York  
10 Columbus Circle  
New York, New York 10019

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Oswego, New York 13126

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Lauman Martin, Esquire  
Senior Vice President  
and General Counsel  
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Staff Coordinator  
New York State Atomic  
Energy Council  
New York State Department  
of Commerce  
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10 Columbus Circle  
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Oswego City Library  
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Oswego, New York 13126

Mr. Robert P. Jones, Supervisor  
Town of Scriba  
R. D. #4  
Oswego, New York 13126



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10  
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Power Authority of the State of New York and Niagara Mohawk Power Corporation (the licensees) dated January 23, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
  
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance:  
January 30, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 10

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Replace pages 181, 182, 183 and 184 of the Appendix A Technical Specifications with the attached pages bearing the same numbers. Changes on these pages are shown by marginal lines.

8. Primary Containment Atmosphere Monitoring Instruments

- a. Primary containment atmosphere shall be continuously monitored when primary containment integrity is required.

B. Standby Gas Treatment System

- 1 Except as specified in 3.7.B.2 below, both circuits of the Standby Gas Treatment System shall be operable at all times when secondary containment integrity is required.

8. Primary Containment Atmosphere Monitoring Instruments

- a. Instrumentation shall be functionally tested and calibrated as specified in Table 4.7-1.

B. Standby Gas Treatment System

1. Standby Gas Treatment System surveillance shall be performed as indicated below:
- a. At least once per operating cycle, it shall be demonstrated that:
- (1.) Pressure drop across the combined high-efficiency and charcoal filters is less than 5.7 in. of water at 6,000 scfm and
  - (2.) 39KW heater outlet shall not have greater than 70 % relative humidity at 6000 scfm.

b. At least once during each scheduled secondary containment leak rate test, whenever a filter is changed, whenever work is performed that could affect the filter system efficiency, and at intervals not to exceed six months between refueling outages, it shall be demonstrated that:

(1.) The removal efficiency of the particulate filters is not less than 99 percent based on a DOP test per ANSI N101.1-1972 para. 4.1.

(2.) The removal efficiency of each of the charcoal filters is not less than 99 percent based on a Freon test.

c. At least once each yr, removable charcoal cartridges shall be removed and absorption capability shall be demonstrated.

d. At least once per operating cycle, automatic initiation of each branch of the Standby Gas Treatment System shall be demonstrated.

2. From and after the date that one circuit of the standby Gas Treatment System is made or found to be inoperable for any reason, the following would apply:
  - a. If in Start-up/Hot Standby, Run or Hot Shutdown mode, reactor operation or irradiated fuel handling is permissible only during the succeeding 7 days unless such circuit is sooner made operable, provided that during such 7 days all active components of the other Standby Gas Treatment Circuit shall be operable.
  - b. If in Refuel or Cold Shutdown mode, reactor operation or irradiated fuel handling is permissible only during the succeeding 31 days unless such circuit is sooner made operable, provided that during such 31 days all active components of the other Standby Gas Treatment Circuit shall be operable.
3. If Specifications 3.7.B.1 and 3.7.B.2 are not met, the reactor shall be placed in the cold condition and irradiated fuel handling operations and operations that could reduce the shutdown margin shall be prohibited.

e. At least once per operating cycle, manual operability of the bypass valve for filter cooling shall be demonstrated.

f. Standby Gas Treatment System Instrumentation Calibration:

differential pressure switches humidity elements and controllers  
Once/operating cycle

2. When one circuit of the Standby Gas Treatment System becomes inoperable, the operable circuit shall be demonstrated to be operable immediately and daily thereafter.

C. Secondary Containment

1. Secondary containment integrity shall be maintained during all modes of plant operation, except when all of the following conditions are met:
  - a. The reactor is subcritical and Specification 3.3.A is met.
  - b. The reactor water temperature is below 212°F, and the Reactor Coolant System is vented.
  - c. No activity is being performed which can reduce the shutdown margin below that specified in Specification 3.3.A.
  - d. The fuel cask or irradiated fuel is not being moved in the reactor building.
2. If Specification 3.7.C.1 cannot be met, procedures shall be initiated to establish conditions listed in Specification 3.7.C.1 within 24 hr.

C. Secondary Containment

1. Secondary containment surveillance shall be performed as indicated below:
  - a. A preoperational secondary containment capability test shall be conducted after isolating the reactor building and placing either Standby Gas Treatment System filter train in operation. Such tests shall demonstrate the capability to maintain a 1/4 in. of water vacuum as indicated by plant instrumentation under calm wind conditions with a filter train flow rate of not more than 6,000 cfm.
  - b. Additional tests shall be performed during the first operating cycle under an adequate number of different environmental wind conditions to enable valid extrapolation of the test results.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

INTRODUCTION

By an application for amendment to Operating License, signed and notarized January 23, 1976, the Power Authority of the State of New York (PASNY) and Niagara Mohawk Power Corporation (NMPC), proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-59, for the James A. FitzPatrick Nuclear Power Plant. The proposed changes involve operability of the Standby Gas Treatment System and Emergency Diesel Generators during fuel handling operations.

DISCUSSION

The Technical Specifications as presently written authorize fuel handling operations to continue only for a seven day period if one circuit of the Standby Gas Treatment System (SGTS) or the pair of emergency diesel generators feeding that circuit of the SGTS are inoperable. The proposed changes would extend the authorization to handle fuel for up to 31 days while one circuit of the SGTS or the on line pair of diesel generators are inoperable.

## EVALUATION

The FitzPatrick Plant is presently shutdown for necessary inspections and modifications related to the local power range monitor instrument tube vibration problem and the low pressure coolant injection (LPCI) system. During the shutdown, as part of the LPCI system modification, each pair of diesels, in turn, will be inoperable. During the shutdown secondary containment integrity is required to be maintained. This in turn requires that one circuit of the SGTS and the pair of emergency diesel generators feeding that circuit remain operable. The necessary work on the diesel generators will exceed the seven day limit presently authorized in the Technical Specifications for one pair of diesels to be inoperable. As a consequence the licensees applied for a Technical Specification change to allow one pair of diesel generators to be inoperable for 31 days. Thirty-one days is the period of inoperability authorized in the G. E. Standard Technical Specifications and in the Technical Specifications of other operating BWR plants. The offsite power sources are necessary for operation during the period of reactor shutdown; thus, in the event of need, one SGTS circuit would have available both an offsite power supply and an emergency power supply and the second SGTS circuit would have an offsite power supply available. In the event a SGTS circuit should become inoperable the existing Technical Specification 4.7.2 provides that the alternate circuit shall be immediately demonstrated to be operable and tested daily thereafter. In the event the alternate circuit cannot be demonstrated operable per T.S. 4.7.2 the fuel handling operation will be terminated per T.S. 3.7.2.b. Due to the availability of offsite power sources to each circuit of the SGTS, the additional availability of one pair of emergency diesel generators and the requirement to terminate fuel handling in the unlikely event of loss of both circuits of the SGTS there is no significant increase in the probability or consequence of a fuel handling accident.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated:

January 30, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-333

POWER AUTHORITY OF THE STATE OF NEW YORK

AND

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. DPR-59 issued to the Power Authority of the State of New York and the Niagara Mohawk Power Corporation which revised the Technical Specifications for operation of the James A. FitzPatrick Nuclear Power Plant, located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specifications to require one circuit of the Standby Gas Treatment System to be operable during fuel handling operations, and a minimum of one pair of emergency diesel generators to be operable when secondary containment integrity is required.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

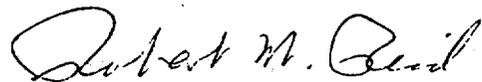
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) application for amendment dated January 23, 1976, (2) Amendment No. 10 to License No. DPR-59, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oswego City Library, 120 East Second Street, Oswego, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 30th day of January, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors