

September 1, 1992

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE
NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M83568)

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated April 21, 1992.

This amendment revises condition 2.C(3) of the license issued December 10, 1984, and removes from Appendix A of that license, fire protection Technical Specifications 1.11, 3.14, 4.15 and 6.1.1(f), Tables 3.14-1 and 3.14-2, and the corresponding section bases for 3.14 and 4.15. The amendment changes and adds to Appendix A of that license, administrative controls by Technical Specifications 6.5.1.6(1) and 6.5.1.7a.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly FEDERAL REGISTER notice.

Sincerely,

Original signed by
Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 49 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDI-3	PM:PDI-3 <i>W</i>	OGC	D:PDI-3 <i>W</i>		
<i>TC</i>	AJohnson:sk	<i>S. Allen</i>	WButler		
<i>8/24/92</i>	<i>08/24/92</i>	<i>8/31/92</i>	<i>9/2/92</i>	<i>1/1</i>	<i>1/1</i>

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

September 21, 1992

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

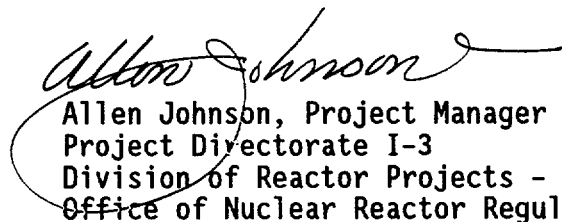
SUBJECT: ISSUANCE OF AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE
NO. DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M83568)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 49 TO FACILITY OPERATION LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated April 21, 1992, Rochester Gas and Electric Corporation (RG&E) requested an amendment to Facility Operation License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. The proposed amendment would remove requirements for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing from the Ginna Technical Specifications (TS) and License as recommended by Generic Letter 86-10 of April 24, 1986. The proposed changes would also modify the administrative control requirements of the TS to add requirements for the Fire Protection Program that are similar to requirements for other programs implemented by licensed condition. Guidance on these proposed changes to the TS was provided to all power reactor licensees and applicants by Generic Letter 88-12, dated August 2, 1988.

One typographical error was corrected in the "Table of Contents" Page i, (Section 4.6) in that "Preferred and" was added to the section title, omitted by the licensee's submittal of April 21, 1992. Minor typographical errors to page 3 of the license have also been corrected. These minor typographical corrections do not change the initial proposed no significant hazards consideration determination.

2.0 DISCUSSION

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Nuclear Regulatory Commission (the Commission) undertook a number of actions to ensure that improvements were implemented in the fire protection programs for all power reactor facilities. A number of revisions and alterations occurred in these programs over the years because of the extensive modification of fire protection programs and the number of open issues resulting from staff evaluations. Consequently, licensees were requested by Generic Letter 86-10 to incorporate the final fire protection program in their Final Safety Analysis Reports (FSARs). In this manner, the Fire Protection Program, including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection, would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the fire protection

program as described in the FSAR, should be used to ensure uniform enforcement of fire protection requirements. The NRC stated that with completion of the requested actions, licensees may request an amendment to delete the fire protection TS, that would be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for a license amendment request to remove fire protection requirements from TS. Additionally, in the licensing review of new plants, the staff had approved applicant requests to remove fire protection requirements from the TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

3.0 EVALUATION

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive fire protection program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in TS can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes. These details, which are presently included in the TS and are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the fire protection program including those technical and administrative requirements removed from the TS to ensure that nuclear safety is not adversely affected. These controls include: (1) the TS administrative controls that are applicable to the fire protection program; (2) the license condition on implementation of, and subsequent changes to, the fire protection program; and (3) the 10 CFR 50.59 criteria for evaluating changes to the fire protection program as described in the FSAR.

The specific details relating to fire protection requirements removed from TS by this amendment include those specifications for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The administrative control requirements for the audit responsibilities of the Plant Operations Review Committee have been expanded by the proposed TS modifications to include the review of the fire protection program and implementing procedures and the submittal of recommended program changes to the Chairman of the Nuclear Safety Audit and Review Board. Under the present TS requirements, fire protection program implementation is listed as an activity of the administrative control requirements for which written procedures shall be established, implemented, and maintained.

The TS changes proposed by the licensee are in accordance with the guidance provided by Generic Letter 88-12, as addressed in the items below.

- (1) Specification 6.5.1.6(1) was revised to add the review of the fire protection program and implementing procedures to the administrative responsibilities of the Plant Operations Review Committee.
- (2) The definition of the fire suppression water system, 1.11, was removed since the term is no longer used to specify TS requirements.
- (3) Specification 3.14.1 and Table 3.14-1 on fire detection instrumentation, Specification 4.15.1 on the associated surveillance requirements, and their bases were removed.
- (4) Specifications 3.14.2 through 3.14.7 and Table 3.14-2 on fire suppression systems and fire barriers, Specifications 4.15.2 through 4.15.7 on the associated surveillance requirements, and their bases were removed.
- (5) Specification 6.2.2 (f) on fire brigade staffing was removed.

Consistent with the guidance of Generic Letter 88-12, the licensee committed to update the FSAR to incorporate the fire protection program either by reference or description. This includes those provisions of fire protection requirements that have been removed from the TS as part of this amendment. Also, the licensee has proposed that the existing license condition on fire protection be replaced with the standard license condition provided in Generic Letter 86-10.

On the basis of its review of this matter, the staff concludes that the licensee has met the guidance of Generic Letter 88-12 for the removal of fire protection requirements from the Ginna TS and has proposed the modification of the license to incorporate the standard condition on fire protection. Therefore, the staff finds the proposed license condition and TS changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements and requirements with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 37572). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed herein, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:

Thomas G. Dunning

Allen R. Johnson

Dated: September 21, 1992

Dr. Robert C. Mecredy

Ginna

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AMENDMENT NO. 49 TO DPR-18 R. E. GINNA NUCLEAR POWER PLANT DATED September 21, 1992

DISTRIBUTION:

Docket File 50-244

NRC & Local PDRs

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated April 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraphs 2.C.(2), and 2.C.(3) of Facility Operating License No. DPR-18 are hereby amended to read as follows*:

*Pages 3 and 4 of Facility Operating License No. DPR-18 are also attached for convenience for the composite license to reflect these changes. Please remove pages 3 and 4 of the existing license and replace with the attached pages.

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 49 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

- (a) The licensees shall implement and maintain in effect all fire protection features described in the licensee's submittals referenced in and as approved or modified by the NRC's Fire Protection Safety Evaluation (SE) dated February 14, 1979 and SE supplements dated December 17, 1980, February 6, 1981, June 22, 1981, February 27, 1985 and March 21, 1985 or configurations subsequently approved by the NRC, subject to provision (b) below.
- (b) The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(c) Deleted

- 3. This license amendment is effective as of its date of issuance, and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

- 1. Changes to the Technical Specifications
- 2. Pages 3 and 4 of Facility Operating License NPR-18

Date of Issuance: September 21, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 49

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the License with the attached pages. The revised pages contain vertical lines indicating the area of change.

Remove

3
4

Insert

3
4

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

i
ii
1-5
3.14-1
3.14-2
3.14-3
3.14-4
3.14-5
3.14-6
3.14-7
3.14-8
3.14-9
3.14-10
4.15-1
4.15-2
4.15-3
4.15-4
4.15-5
6.2-3
6.5-4

Insert

i
ii .
1-5
3.14-1

4.15-1

6.2-3
6.5-4

- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
 - (1) Maximum Power Level

RG&E is authorized to operate the facility at steady-state power levels up to a maximum of 1520 megawatts (thermal).
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Fire Protection
 - (a) The licensee shall implement and maintain in effect all fire protection features described in the licensee's submittals referenced in and as approved or modified by the NRC's Fire Protection Safety Evaluation (SE) dated February 14, 1979 and SE supplements dated December 17, 1980, February 6, 1981, June 22, 1981, February 27, 1985 and March 21, 1985 or configurations subsequently approved by the NRC, subject to provision (b) below.
 - (b) The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(c) Deleted

(4) Secondary Water Chemistry Monitoring Program

The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall be described in the plant procedures and shall include:

- (a) Identification of a sampling schedule for the critical parameters and control points for these parameters;
- (b) Identification of the procedures used to measure the values of the critical parameters;
- (c) Identification of process sampling points;
- (d) Procedure for the recording and management of data;
- (e) Procedures defining corrective actions for off control point chemistry conditions; and
- (f) A procedure identifying (i) the authority responsible for the interpretation of the data, and (ii) the sequence and timing of administrative events required to initiate corrective action.

(5) Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as reasonably achievable levels. This program shall include the following:

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1.10 Hot Channel Factors

F_Q , Heat Flux Hot Channel Factor, is defined as the maximum local heat flux on the surface of a fuel rod divided by the average fuel rod heat flux allowing for manufacturing tolerances on fuel pellets and rods.

F_Q^N , Nuclear Heat Flux Hot Channel Factor, is defined as the maximum local fuel rod linear power density divided by the average fuel rod linear power density assuming nominal fuel pellet and rod dimensions.

F_Q^E , Engineering Heat Flux Hot Channel factor, is defined as the ratio between F_Q and F_Q^N and is the allowance on heat flux required for manufacturing tolerances.

$F_{\Delta H}^N$, Nuclear Enthalphy Rise Hot Channel Factor, is defined as the ratio of the integral of linear power along the rod on which minimum DNBR occurs to the average rod power.

1.11 (DELETED)

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f. (DELETED)

- g. Adequate shift coverage shall be maintained without routine heavy use of overtime. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions including senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel. Changes to the guidelines for the administrative procedures shall be submitted to the NRC for review.

RESPONSIBILITIES (Continued)

- h. Review of the Plant Security Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.
- i. Review of the Radiation Emergency Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.
- j. Review of implementing procedures for the Plant Security Plan and the Radiation Emergency Plan and proposed changes thereto.
- k. Review of all Reportable Events.
- l. Review of the Fire Protection Program and Implementing Procedures and submittal of recommended Program changes to the Chairman of the Nuclear Safety Audit and Review Board (NSARB).

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend in writing to the Plant Manager, Ginna Station approval or disapproval of items considered under 6.5.1.6(a) through (d) and (1) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (d) and (1) above constitutes an unreviewed safety question as defined in 10 CFR Section 50.59.