

August 8, 1986

Docket No. 50-244

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649-0001

Dear Mr. Kober:

The Commission has issued the enclosed Amendment No. 16 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated January 6, 1986, which effectively supersedes your application dated October 16, 1985.

The amendment changes the Technical Specifications in regard to facility and corporate organization, changes management titles and enables the formation of a new facility operating group. This action completes our TAC No. 59831.

A copy of the Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/s/
Morton B. Fairtile, Project Manager
Project Directorate #1
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 16 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

*SEE PREVIOUS CONCURRENCE

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|----------|--------------------------|---------------|----------|------------------------------|
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| Surname: | PShuttleworth | *MFairtile/ | *MWagner | GLear <i>by MD J</i> |
| Date: | <i>8/8/86</i> <i>MLL</i> | 07/21/86 | 07/29/86 | <i>07/08/86</i> <i>08</i> |

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Mr. Roger W. Kober
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 16
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated October 16, 1985 and January 6, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 16, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Morton B. Fairtile

Morton B. Fairtile, Project Manager
Project Directorate #1
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 8, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 16

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

6.1-1
6.2-3
6.2-4
6.2-5
6.2-6
6.2-7
6.5-1
6.5-2
6.5-11

INSERT

6.1-1
6.2-3
6.2-4
6.2-5
6.2-6
6.2-7
6.5-1
6.5-2
6.5-11

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Superintendent-Ginna Production shall be responsible for overall on-site Ginna Station operation and shall delegate in writing the succession to this responsibility during his absence.

—— LINES OF SUPERVISION AND ADMINISTRATION
 - - - LINES OF COMMUNICATION

R. E. GINNA NUCLEAR POWER PLANT MANAGEMENT ORGANIZATION CHART

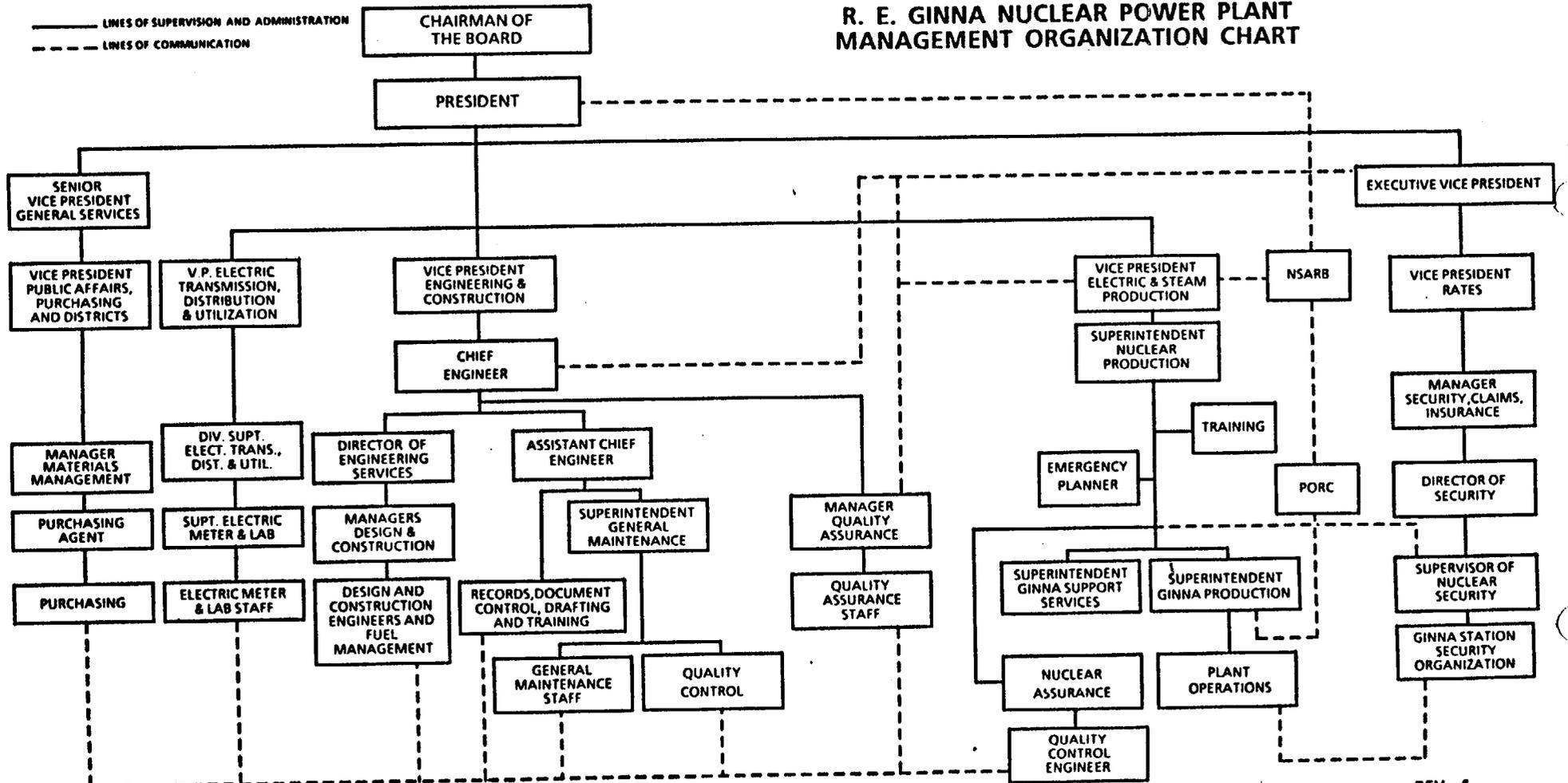
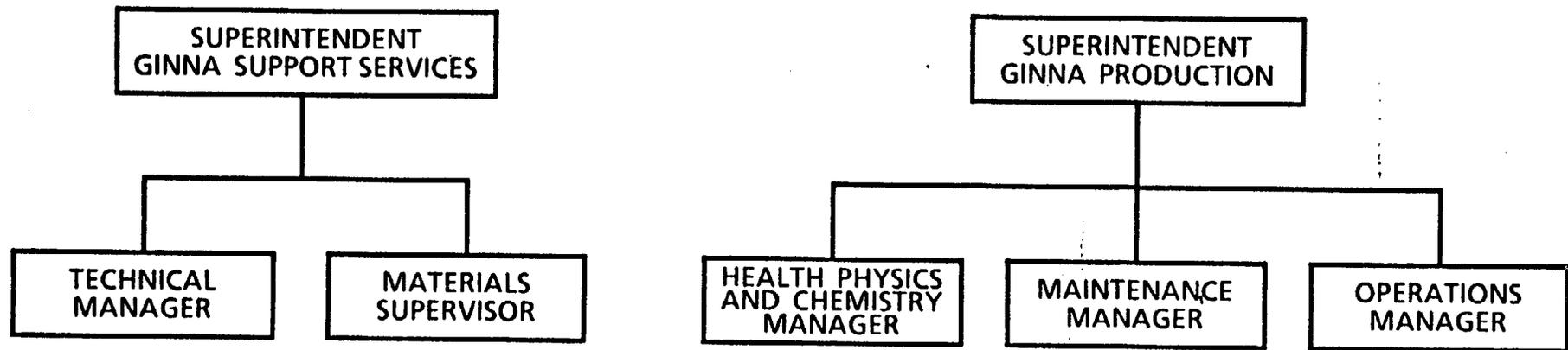


FIGURE 6.2-1

REV. 4

ROCHESTER GAS AND ELECTRIC CORPORATION GINNA STATION ORGANIZATION



6.2-4

Figure 6.2-2

**R. E. GINNA STATION
ADMINISTRATIVE AND HEALTH PHYSICS & CHEMISTRY SECTION
GROUP ORGANIZATIONS**

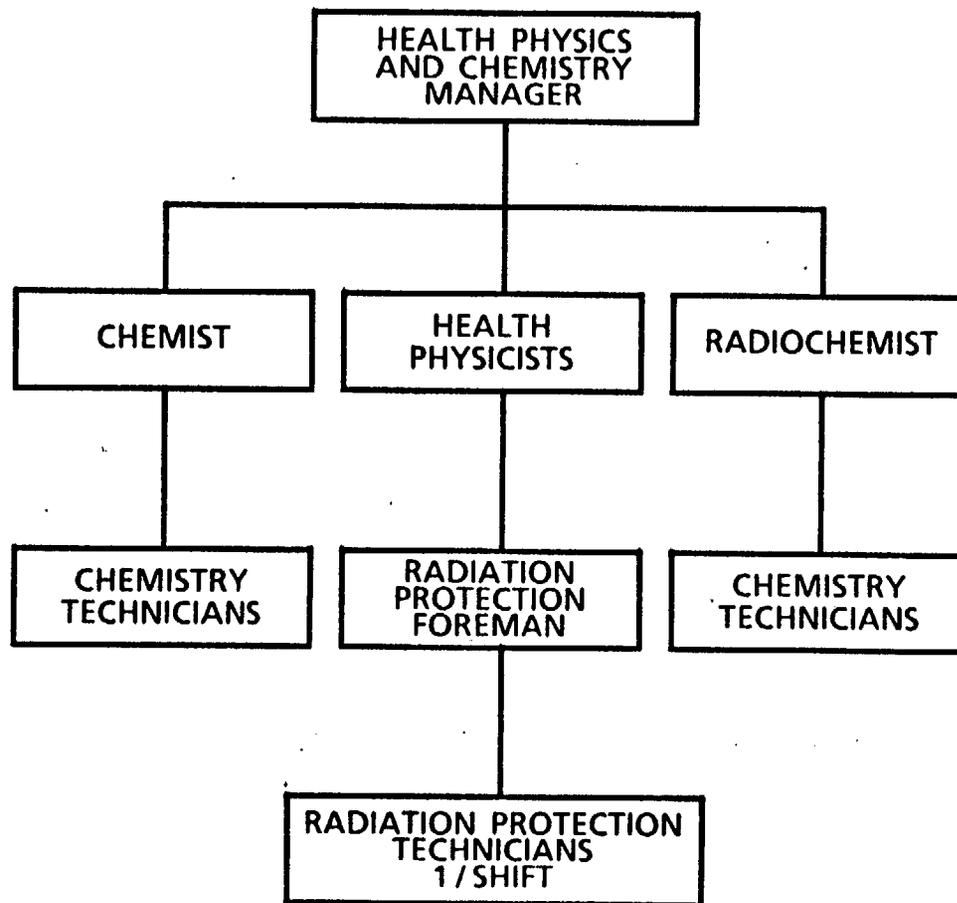


Figure 6.2-3

R. E. GINNA STATION MAINTENANCE AND NUCLEAR ASSURANCE SECTION GROUP ORGANIZATIONS

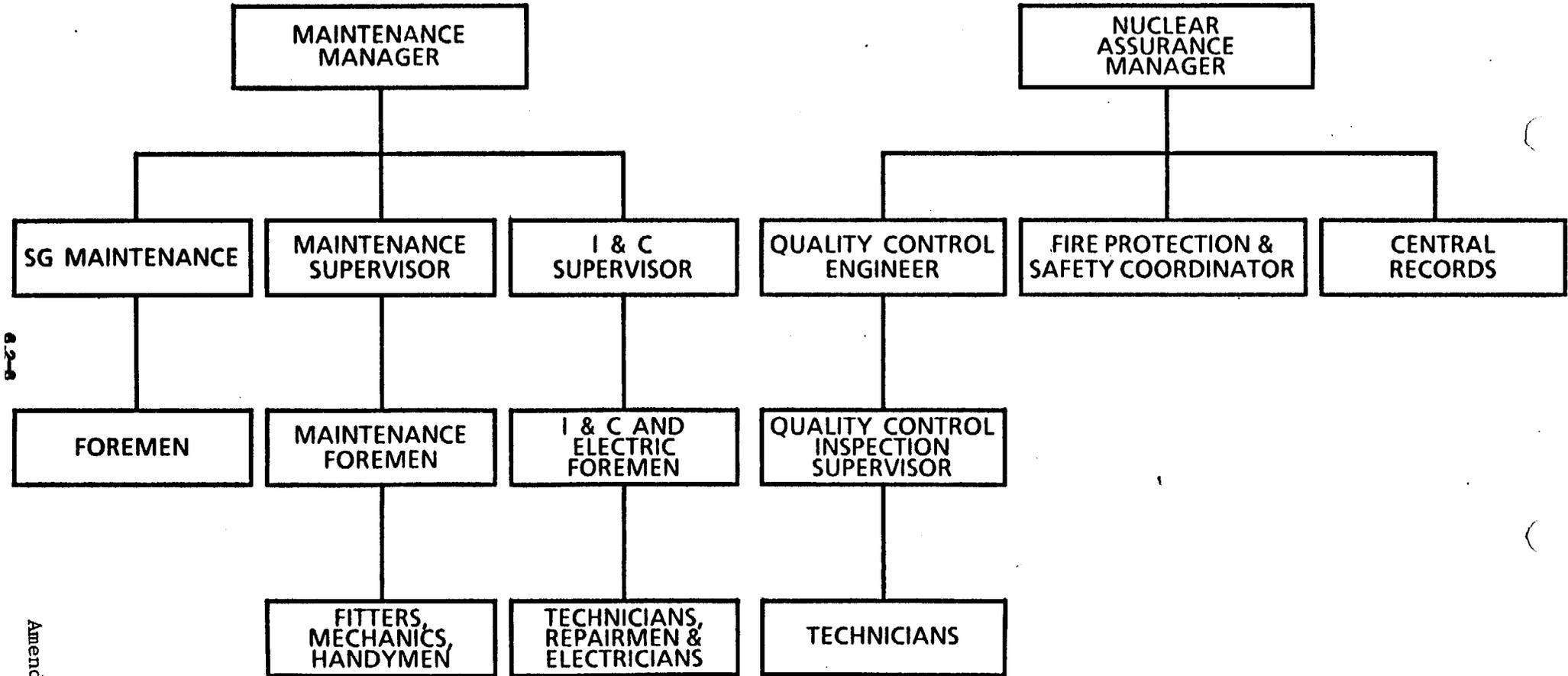


Figure 6.2-4

6.2-8

R. E. GINNA STATION OPERATIONS AND TECHNICAL SECTION GROUP ORGANIZATIONS

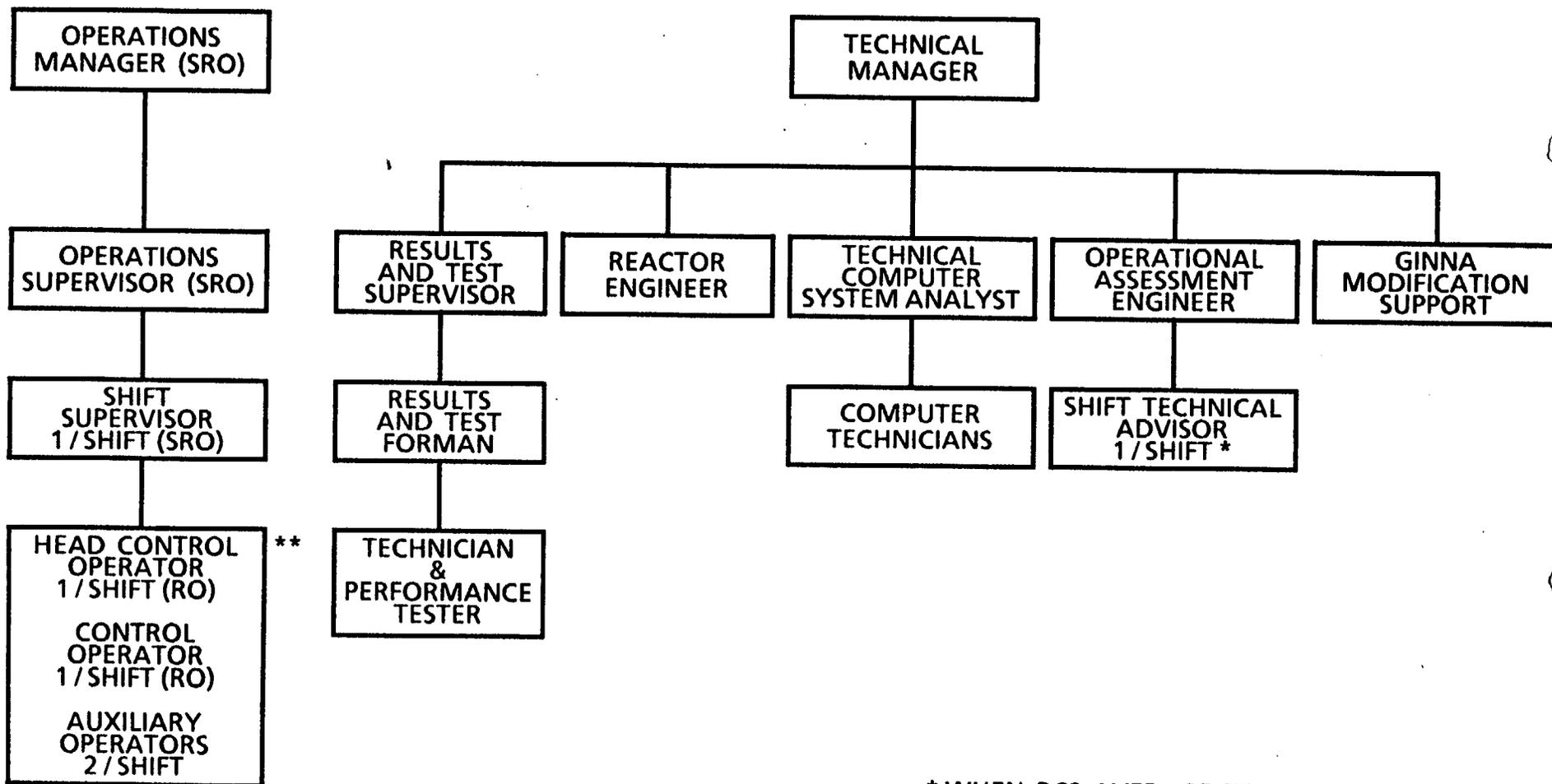


Figure 6.2-5

* WHEN RCS AVERAGE TEMPERATURE >200°F

** A SECOND SRO WILL BE ON SHIFT AS REQUIRED BY 10 CFR 50.54 WHEN RCS AVERAGE TEMPERATURE >200°F

6.7 REVIEW AND AUDIT

Three separate organizational units shall be established for the purpose of review and audit of plant operations and safety-related matters. One of these will be an on-site operations review group, the Plant Operations Review Committee (PORC). A second is the Quality Assurance (Q.A.) group, responsible for the audit of safety-related activities associated with plant operations. A third is the independent audit and review group, the Nuclear Safety Audit and Review Board (NSARB). This group is responsible for the periodic review of the activities of the Plant Operations Review Committee, for directing audits and evaluating their results, and for the management evaluation of the status and adequacy of the quality assurance program.

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC) FUNCTION

6.5.1.1 The Plant Operations Review Committee shall function to advise the Superintendent - Ginna Production on all matters related to nuclear safety and for referral of appropriate matters to the Nuclear Safety Audit and Review Board.

COMPOSITION

6.5.1.2 The PORC shall be composed of the;

Chairman: Superintendent - Ginna Production

Vice Chairman/Member: Superintendent - Ginna Support
Services

Vice Chairman/Member: Operations Manager

Member: Maintenance Manager

Member: Instrument & Control Supervisor

Member: Technical Manager

Member: Results and Test Supervisor

Member: Reactor Engineer

Member: Health Physics and Chemistry Manager

Member: Nuclear Assurance Manager

Member: Quality Control Engineer

ALTERNATES

6.5.1.3 Alternate members shall be designated by name, in writing by the Chairman.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

QUORUM

6.5.1.5 A quorum of the PORC shall consist of the Chairman or Vice Chairman and four members including alternates. No more than two shall be alternates.

AUTHORITY

- 6.5.2.9 a. The chairman of the Nuclear Safety Audit and Review Board is responsible to the President on all activities for which the review board is responsible.
- b. The NSARB shall report to and advise the Vice President, Electric and Steam Production, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of NSARB activities shall be prepared, approved, and distributed as indicated below:
- a. Minutes shall be recorded of all meetings of this Board. Copies of the minutes shall be forwarded within 14 days following each meeting to the Corporate Chairman of the Board, Vice President, Electric and Steam Production and such others as the Chairman of the NSARB may designate.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g, and h above, shall be prepared, approved and forwarded to the Vice President-Electric and Steam Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Electric and Steam Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 16 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

INTRODUCTION

By letters dated October 16, 1985 and January 6, 1986, the Rochester Gas and Electric Corporation (RG&E or the licensee) submitted a proposed license amendment for Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant (the facility). The January 6th submittal completely replaced the October 16th submittal; therefore, the following evaluation is of the January 6, 1986 submittal. The license amendment changes the Technical Specifications (TS) in regard to facility and corporate organization, changes management titles and enables the formation of a new facility operating group.

EVALUATION

RG&E submitted this application to change the TS based on changes to the corporate management organization brought about by the sudden death of the Senior Vice President, Operations. Previously, Vice Presidents reported to the Senior Vice President, Operations, now they report directly to the President as the position of Senior VP, Operations has been removed from the corporate management organization. This change shortens the reporting path from the Manager, Quality Assurance to the upper corporate management levels and results in an increase in corporate attention to quality assurance. In addition, the Quality Assurance (QA) organization will now be a responsibility of the Executive Vice President. This change will make the QA organization independent of the design, construction and other engineering functions. We conclude that the corporate organizational changes as related to the QA organization provide more independence to the QA organization and also provide more direct upper corporate level management involvement and are therefore acceptable.

The plant organization responsible for quality programs is the Nuclear Assurance Group.

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The roles of the above organizations, Quality Assurance and Nuclear Assurance, are as follows. The QA organization is a corporate organization responsible for establishing and assuring implementation of a quality program that meets the requirements of 10 CFR 50 Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants." The Nuclear Assurance organization is responsible for coordinating the implementation of the quality program at the station and is independent of the QA organization and will report to the Superintendent, Nuclear Production. However, the QA organization will monitor program implementation through review of station procedures and related audits. We conclude that the portion of the reorganization related to QA and Nuclear Assurance will provide greater management attention to plant quality programs and is therefore acceptable.

There are other changes in the plant organization. The Superintendent, Nuclear Production, the equivalent to a plant manager's position, now reports directly to the Vice President, Electric and Steam Production. Two new positions have been created, that of Superintendent-Ginna Production and Superintendent-Ginna Support Services, who report to the Superintendent, Nuclear Production. The new Plant Operations organization reports directly to the Superintendent-Ginna Production. The position of Assistant Superintendent has been eliminated. The Health Physics and Chemistry Manager, the Maintenance Manager, and the Operation Manager will report to the Superintendent-Ginna Production. The Technical Manager and the Materials Supervisor will report to the Superintendent-Ginna Support Services. We conclude that these changes will make the day-to-day operation of the station more effective by creating a group to concentrate on operations, maintenance and health physics and chemistry and are therefore acceptable.

OVERALL CONCLUSION ON REORGANIZATION

Based on the above, we conclude that the reorganization will increase management attention regarding the operation of the plant, will increase effectiveness of the plant staff and enhance both corporate level and plant quality programs and therefore the reorganization is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to requirements with respect to Administrative Controls. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(7). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

ACKNOWLEDGEMENT

Principal Contributors: M. B. Fairtile

Dated: August 8, 1986

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