

December 29, 1992

Docket No. 50-244

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Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO.
DPR-18, R. E. GINNA NUCLEAR POWER PLANT (TAC NO. M84742)

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated June 22, 1992.

This amendment would change Technical Specifications by removing the schedule for the withdrawal of Reactor Vessel Material Specimens from Technical Specification Surveillance Requirement 4.3.1 in accordance with the guidance described in the proposed Specification 4.3.1.1. In addition, RG&E shall maintain the Commission's approved version of the specimen withdrawal schedule in the Updated Final Safety Analysis Report (UFSAR). The addition of this schedule will be in the next revision of the USFAR in accordance to 10 CFR 50.71(e).

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDI-3	PM:PDI-3 <i>[initials]</i>	PM:PD1-3 <i>[initials]</i>	OGC <i>[initials]</i>	D:PDI-3 <i>[initials]</i>	
TCClark <i>[initials]</i>	RSkokowski	AJohnson	<i>E. Holler</i>	WButler <i>[initials]</i>	
11/12/92	11/12/92	11/12/92	11/14/92	12/21/92	1/1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 29, 1992

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President, Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

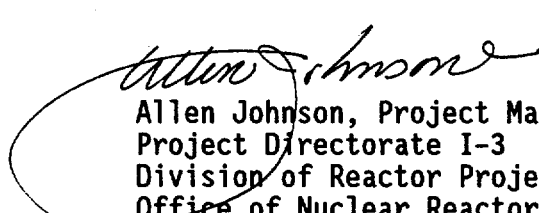
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Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to
License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

Dr. Robert C. Mecredy

Ginna

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated June 22, 1992 complies with the standard and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Walter R. Butler".

Walter R. Butler, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 29, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

4.3-1
4.3-1a

Insert

4.3-1

4.3 Reactor Coolant System

Applicability

Applies to surveillance of the reactor coolant system and its components.

Objective

To ensure operability of the reactor coolant system and its components.

Specifications:

4.3.1 Reactor Vessel Material Surveillance Testing

4.3.1.1 The reactor vessel material surveillance specimens shall be removed and examined to determine changes in their material properties, as required by Appendix H to 10 CFR Part 50.

4.3.2 Pressurizer

4.3.2.1 The pressurizer water level shall be verified to be within its limits at least once per 12 hours during power operation and hot shutdown.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated June 22, 1992, the Rochester Gas and Electric Corporation (the licensee) requested an amendment to Facility Operating License No. DPR-18 to change the technical specifications (TS) for the R. E. Ginna Nuclear Power Plant. The proposed change removes TS 4.3.1.2 and the table that provides the schedule for reactor vessel material specimen withdrawal, which is a part of TS 4.3.1.1. Guidance on the proposed TS change was provided by Generic Letter (GL 91-01) dated January 4, 1991, to all holders of operating licenses or construction permits for nuclear power reactors.

2.0 EVALUATION

TS 3.1.2, "Heatup and Cooldown Limit Curves for Normal Operation," contains the limiting condition for operation for the Reactor Coolant System (RCS) that limits the rate of change in temperature and pressure to values consistent with the fracture toughness requirements of the American Society of Mechanical Engineers (ASME) Code and Appendix G to Part 50 of Title 10 of the Code of Federal Regulations (10 CFR Part 50). Changes in the values of these limits are necessary because the fracture toughness properties of ferritic materials in the reactor vessel change as a function of the reactor operating time (neutron fluence).

For this reason, the TS includes a surveillance requirement, TS 4.3.1, "Reactor Vessel Material Surveillance Testing," to require the removal and examination of the irradiated specimens of reactor vessel material. The licensee examines the specimens to determine the changes in material properties in accordance with the requirements of Appendix H to 10 CFR Part 50. TS 4.3.1.1 includes a table that identifies the material specimens and specifies the schedule for removal of each specimen. TS 4.3.1.2 states that the report of the reactor vessel material surveillance shall be a summary technical report as required by Appendix H to 10 CFR Part 50.

The removal of the schedule for withdrawing material specimens from the TS will eliminate the necessity of a license amendment to make changes to this schedule. However, Section I.B.3 of Appendix H to 10 CFR Part 50 requires the submittal of a proposed withdrawal schedule for material specimens to the U.S.

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Nuclear Regulatory Commission (NRC) and approval by the NRC before implementation. Hence, adequate regulatory controls exist to control changes to this schedule without the necessity of subjecting it to the license amendment process by including it in the TS. Likewise, the requirements of TS 4.3.1.2 are redundant to the requirements of Appendix H to 10 CFR Part 50 and are, therefore, unnecessary.

The licensee has provided a commitment to include this schedule in the next revision of the Updated Final Safety Analysis Report (UFSAR). In addition, the licensee will include any subsequent NRC-approved revisions to this schedule in an update of the UFSAR. The inclusion of the withdrawal schedule in the UFSAR provides a source for this information that is readily available as a reference for NRC inspectors and other staff use. Finally, the surveillance requirements for removing material specimens and the bases section for this specification remain unchanged.

The licensee has proposed a change to TS 4.3.1.1 and the removal of TS 4.3.1.2 that is consistent with the guidance provided in Generic Letter 91-01 for the removal of the table of reactor vessel material specimen withdrawal schedule from the TS. The NRC has reviewed this matter and finds that the proposed changes to the Ginna TS are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State Official was notified of the proposed issuance of the amendment. The State Official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (57 FR 55589). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:
Thomas G. Dunning
Allen R. Johnson

Date: December 29, 1992